



Pennsylvania Power & Light Company

Two North Ninth Street • Allentown, PA 18101-1179 • 610/774-5151

Robert G. Byram
Senior Vice President-Nuclear
610/774-7502
Fax: 610/774-5019

AUG 07 1996

U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Mail Station P1-137
Washington, DC 20555

**SUSQUEHANNA STEAM ELECTRIC STATION
RWCU REGENERATIVE HEAT EXCHANGER REPAIR
PLA-4487**

FILE R41-2

Docket No. 50-387/NPF-14

Reference: PLA-4452, Request to Perform Temporary Non-Code Repairs to the RWCU Heat Exchanger, dated April 23, 1996.

On April 23, 1996 PP&L requested relief from performing ASME Code Section XI repairs on the SSES Unit 1 Reactor Water Cleanup Regenerative Heat Exchanger 1E207A (PLA-4452). Relief was granted in your letter of April 29 1996, (Stolz to Byram, Docket 50-387). The repair has been successfully implemented.

Upon subsequent review of your letter granting approval to implement the repair, a concern arose on our part regarding statements made related to the repair configuration. Your letter stated that the repair would reduce the stresses on the flange bolts. In our review, we concluded that the repair would have a negligible effect on bolt stresses, but we could not conclude that the bolt stresses would be reduced. The following description is provided for your information to assure understanding of the installed repair configuration.

The steam leak repair implemented for the RWCU regenerative heat exchanger 1E207A involved use of a clamp/nut enclosure system (hereafter "enclosure system") with compound injection. The enclosure system was clamped to be in-contact with the circumferential surface of the heat exchanger channel cover and the circumferential surface of the heat exchanger channel flange.

The enclosure system was installed to form the space in which the sealing compound would be injected. The space was formed by the enclosure inner surface, the channel cover inner surface (surface facing the heat exchanger), the channel outward facing flange surface, and the diaphragm circumference surface. The compound was injected into this space. This

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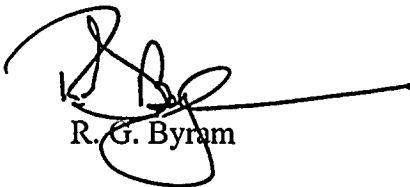


configuration reduces the pre-load on the flange joint, but it would have no impact on the bolt stresses until the joint pre-load was completely relieved. The installation load never reached the point where the joint pre-load stresses would have been completely relieved.

A permanent ASME Code repair will be implemented during the next refueling outage, scheduled to begin in September 1996. Closure of all actions relevant to the repair are being tracked under PP&L's Corrective Action Program. In addition, as requested in your letter, a structural assessment of the effects of the repair on the heat exchanger has been performed. The assessment concluded that the effects of the repair on the heat exchanger are negligible.

If you have any questions on this issue, please contact Mr. C. T. Coddington at (610) 774-7531.

Very truly yours,



R. G. Byram

copy: Regional Administrator - Region I
Mr. K. Jenison, NRC Sr. Resident Inspector
Mr. C. Poslušny, Jr., NRC Sr. Project Manager



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