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SUBJECT: Documents info verbally provided to NRC staff on 960402 re
Unit 2 core shroud insp analysis, as provided in ltr dtd
951025.

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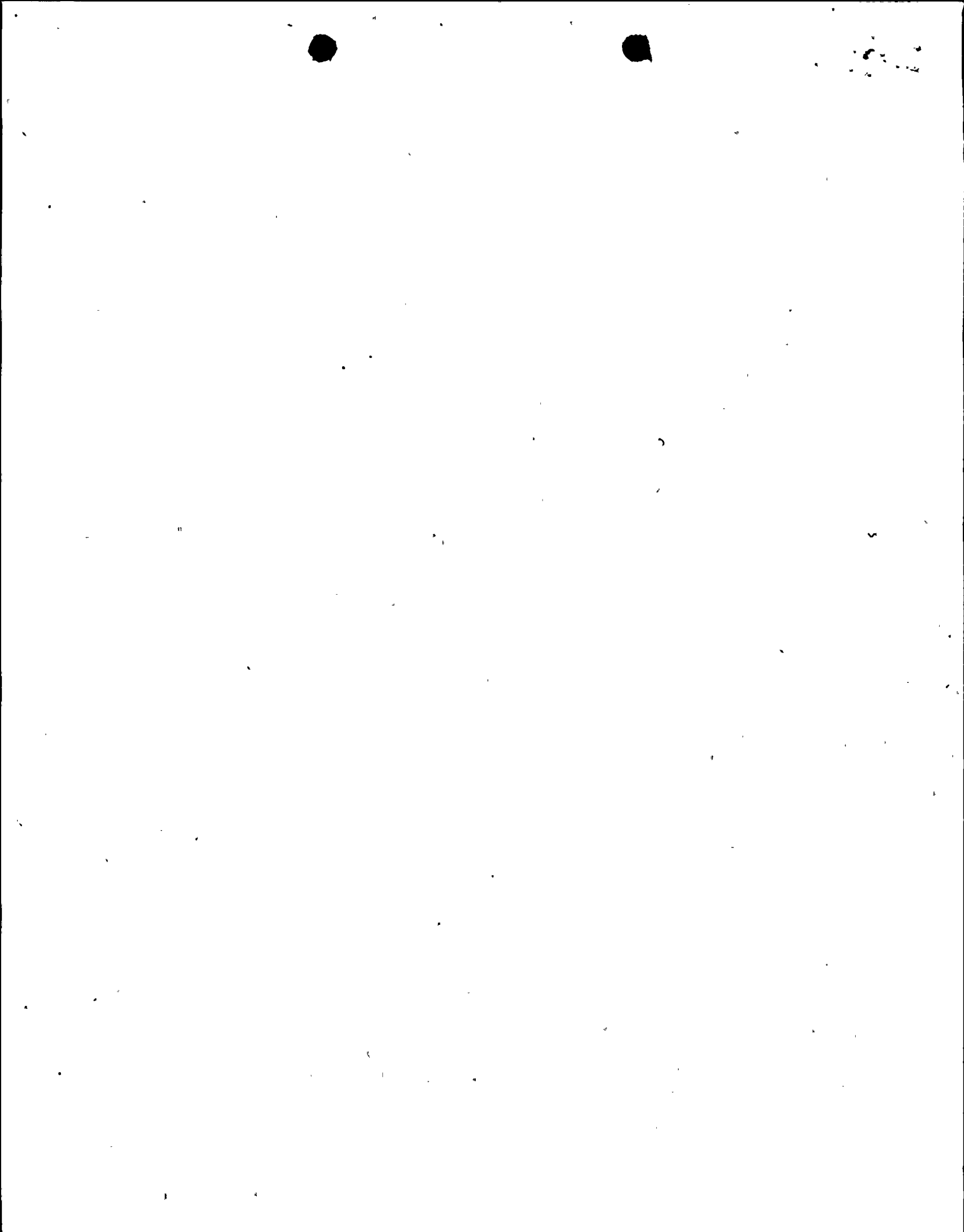
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**SUSQUEHANNA STEAM ELECTRIC STATION
RESPONSE TO VERBAL RAI PERTAINING TO
UNIT 2 CORE SHROUD ANALYSIS**

PLA-4445

R41-2A

Docket No. 50-388

Reference: PLA-4382 from R. G. Byram to USNRC, "Generic Letter 94-03: Final Inspection Results for Unit 2 Core Shroud," dated October 25, 1995.

This letter documents information that was verbally provided to the NRC staff on April 2, 1996, concerning the Unit 2 Core Shroud inspection analysis as provided in the referenced letter.

NRC Information Request #1: Was the crack growth calculations applied to both ends of the defects?

PP&L Response: Crack growth and detection errors were applied to both ends of the defect for the evaluation of the integrity of the remaining material.

NRC Information Request #2: Was the analysis performed for the Unit 2 core shroud the same as for Unit 1?

PP&L Response: The evaluation for the Unit 2 core shroud utilized the same methodology used for Unit 1. The only difference was in the application of the uncertainty factors to the detection of defects using the GE Tracker system. The factors used for Unit 2 were unavailable for the Unit 1 analysis; therefore, generic/conservative numbers, as found in reference 6.3 of the "BWR Core Shroud Inspection and Flaw Evaluation Guidelines," were used. The numbers for the Unit 2 shroud evaluation were obtained from reference 6.5 of the BWRVIP September 15, 1995, draft report "Reactor Pressure Vessel and Internals Examination Guidelines," and represent more realistic values that have been verified by actual test results. These factors and the use of them were approved by the BWRVIP and its member utilities, including PP&L, and issued as EPRI document TR-105696, dated October 1995.

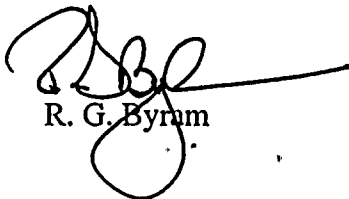
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If you have additional questions, please contact R. D. Kichline at 610-774-7705.

Very truly yours,



R. G. Byram

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