

PRIORITY 1

(ACCELERATED RIDS PROCESSING)

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 9511270350 DOC. DATE: 95/11/20 NOTARIZED: NO DOCKET #
 FACIL: 50-388 Susquehanna Steam Electric Station, Unit 2, Pennsylvania 05000388
 AUTH. NAME AUTHOR AFFILIATION
 CODDINGTON, C.T. Pennsylvania Power & Light Co.
 STANLEY, H.G. Pennsylvania Power & Light Co.
 RECIP. NAME RECIPIENT AFFILIATION

SUBJECT: LER 95-014-00: on 951020, unit manually scrambled. Caused by failure of acoustic monitor due to failure of amphenol connector. Failed amphenol connector replaced & other amphenol connectors inspected. W/951120 ltr.

DISTRIBUTION CODE: IE22T COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 4
 TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc.

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EXTERNAL:	L ST LOBBY WARD		1	1	LITCO BRYCE, J H		2	2
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Pennsylvania Power & Light Company

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November 20, 1995

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

SUSQUEHANNA STEAM ELECTRIC STATION
LICENSEE EVENT REPORT 95-014-00
PLAS - 656 FILE R41-2

Docket No. 50-388
License No. NPF-22

Attached is Licensee Event Report 50-388/95-014-00. This event was determined to be reportable per 10CFR50.73(a)(2)(i) in that the Susquehanna Steam Electric Station - Unit 2 completed a reactor shutdown as required by the unit's Technical Specifications due to the failure of the acoustic monitor for the 'L' Main Steam Safety/Relief Valve during the performance of surveillance testing.


H.G. Stanley
VP - Nuclear Operations

CTC/dmd

cc: Mr. T. T. Martin
Regional Administrator, Region I
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Berwick, PA 18603-0035

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LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Susquehanna Steam Electric Station - Unit 2	DOCKET NUMBER(2) 0 5 0 0 0 3 8 8 1	PAGE (3) OF 0 3
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TITLE (4)
Technical Specification Required Shutdown

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)															
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)														
1	0	2	0	9	5	9	5	0	1	4	0	0	1	1	2	0	9	5		0	5	0	0	0

OPERATING MODE (9) 2	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)									
POWER LEVEL (10) 0 0 8	<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.405(c)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(b)						
	<input type="checkbox"/> 20.405(a)(1)(i)	<input type="checkbox"/> 50.36(c)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(c)						
	<input type="checkbox"/> 20.405(a)(1)(ii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(v)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)						
	<input type="checkbox"/> 20.405(a)(1)(iii)	<input checked="" type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(v)(A)							
	<input type="checkbox"/> 20.405(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(1)(2)(v)(B)							
<input type="checkbox"/> 20.405(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(v)								

(LICENSEE CONTACT FOR THIS LER (12))

NAME Cornelius T. Coddington - Sr. Project Engineer, Nuclear Licensing	TELEPHONE NUMBER
	AREA CODE: 7 1 7 NUMBER: 5 4 2 - 3 2 8 9

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)

<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On October 19, 1995, at 2025 hours with Unit 2 starting up in Condition 2 at 0% power, the acoustic monitor for the 'L' Main Steam Safety/Relief Valve (SRV) was declared inoperable and Technical Specifications 3.3.7.5, action 80b, and 3.4.2, action c, were entered. Repair of acoustic monitor required a primary containment entry; therefore, the unit was manually scrammed on October 20, 1995, at 1619 hours from 8% power. All control rods inserted fully. There were no SRV lifts, Emergency Core Cooling System (ECCS) injections, or diesel generator starts as a result of the manual scram. The shutdown of the unit is reportable per 10CFR50.73(a)(2)(i) in that the Susquehanna SES Unit 2 completed a reactor shutdown as required by the unit's Technical Specifications. The failure of the acoustic monitor was due to a broken wire in the Amphenol Connector. The connector was replaced and other connectors were inspected.

**LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION**

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Unit 2 Susquehanna Steam Electric Station	DOCKET NUMBER (2) 0 5 0 0 0 3 8 8	LER NUMBER (6)						PAGE (3)		
		YEAR		SEQUENTIAL NUMBER		REVISION NUMBER				
		9 5	—	0 1 4	—	0 0	2	OF	3	

TEXT (If more space is required, use additional NRC Form 366A's) (17)

EVENT DESCRIPTION

On October 19, 1995, at 2025 hours with Unit 2 starting up in Condition 2 at 0% power, the acoustic monitor for the 'L' Main Steam Safety/Relief Valve (SRV) (EIS Code: SB) was declared inoperable during the performance of surveillance testing. While performing the valve closure steps of the surveillance procedure, the acoustic monitor lights for the 'L' SRV remained illuminated, indicating the SRV was open. The off-normal procedure was immediately performed and alternate control room indications were used to confirm the SRV was closed. The SRV tailpipe temperature trend indicated a downward trend and reactor pressure remained steady. Technical Specifications 3.3.7.5, action 80b, and 3.4.2, action c, were entered.

Repair of the monitor required a primary containment entry; therefore, the unit was manually scrammed on October 20, 1995, at 1619 hours from 8% power. All control rods inserted fully. There were no SRV lifts, ECCS injections, or diesel generator starts as a result of the manual scram. The shutdown of the unit is reportable per 10CFR50.73(a)(2)(i) in that the Susquehanna SES Unit 2 completed a reactor shutdown as required by the unit's Technical Specifications.

CAUSE OF EVENT

The failure of the acoustic monitor was due to the failure of the Amphenol Connector. The failure of the connector was due to a broken wire resulting from stress put on the cable. However, there were several problems with the connector. In addition to the broken wire, there was a missing screw on the cable connector. The screw missing from the cable connector leaves the cable wiring susceptible to damage because the screw holds the cable tight in the connector.

REPORTABILITY/ANALYSIS

This event was determined to be reportable per 10CFR50.73(a)(2)(i) in that the Susquehanna SES Unit 2 completed a reactor shutdown as required by the unit's Technical Specifications. Technical Specifications 3.3.7.5 and 3.4.2 require all Main Steam Safety/Relief Valve acoustic monitors to be operable in operational conditions 1 and 2. Since the acoustic monitor could not be restored to operable status within the time frame specified in the Technical Specifications, the unit was shutdown as required by the Technical Specifications.

**LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION**

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION
REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE
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(3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

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		9 5 —	0 1 4 —	0 0	3	OF	3			

TEXT (If more space is required, use additional NRC Form 366A's) (17)

The failure of the acoustic monitor does not affect the safety function of the SRV. The position of the SRV can be confirmed by indications other than the acoustic monitor. These indications included SRV tailpipe temperature and reactor pressure. The loss of the acoustic monitor does not affect the health and safety of the public.

In accordance with the guidance provided in NUREG 1022, Supplement 1, Item 14.1 and 10CFR50.4(d), the required submission date for this report was determined to be November 20, 1995.

CORRECTIVE ACTIONS

Upon receipt of the indication that the SRV was open, the off-normal procedure was immediately performed and alternate control room indications were used to confirm the SRV was closed. The acoustic monitor for the 'L' Main Steam Safety/Relief Valve was declared inoperable and Technical Specifications 3.3.7.5, action 80b, and 3.4.2, action c, were entered. An investigation revealed that a broken wire in the Amphenol Connector caused the failure of the acoustic monitor. In addition to the failed Amphenol Connector having been replaced, the other Amphenol Connectors were inspected and verified to operate properly.

The following corrective actions are being performed:

- Appropriate procedure will be revised to incorporate instructions which address requirements for connector inspections.
- Engineering will evaluate possible alternatives to enhance the existing cable/connector arrangement.

ADDITIONAL INFORMATION

Past Similar Events: Docket No. 50-388 LER 94-003-00

Failed Component: None

