

PRIORITY 1

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REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 9510230012 DOC. DATE: 95/10/17 NOTARIZED: NO DOCKET #
FACIL: 50-388 Susquehanna Steam Electric Station, Unit 2, Pennsylv 05000388
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RECIP. NAME RECIPIENT AFFILIATION

SUBJECT: LER 95-011-00: on 950920, MS line penetration leakage rate exceeded TS limit. Stroked "B" & "D" inboard & "A" outboard MSL MSIVs & reperformed MSIV LLRT.W/951017 ltr.

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TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc.

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October 17, 1995

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

SUSQUEHANNA STEAM ELECTRIC STATION
LICENSEE EVENT REPORT 95-011-00
PLAS - 649 FILE R41-2

Docket No. 50-388
License No. NPF-22

Attached is Licensee Event Report 95-011-00. This event was determined to be reportable per 10CFR50.73(a)(2)(ii) in that the Main Steam Line penetration leakage exceeded the Technical Specification limit during regularly scheduled Local Leak Rate Testing.


H.G. Stanley
VP - Nuclear Operations

CTC/dmd

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**LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION**

FACILITY NAME (1) Unit 2 Susquehanna Steam Electric Station	DOCKET NUMBER (2) 0 5 0 0 0 3 8 8	LER NUMBER (6)	PAGE (3)									
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TEXT (If more space is required, use additional NRC Form 366A's) (17)

EVENT DESCRIPTION

At 0415 hours on September 20, 1995, with Unit 2 in its seventh refueling and inspection outage (Condition 5, Refueling, 0% power), an evaluation of data from the scheduled Main Steam (MSL, EIS Code: SB) penetration Local Leak Rate Tests (LLRTs) determined that the "as found" leakage through both the inboard and outboard Main Steam Isolation Valves (MSIVs, EIS Code: BD) was in excess of the limit of Technical Specification 3.6.1.2.c for the total MSL containment penetration leakage of 21.7 standard liters per minute (slm) (46.0 SCFH). The total "as found" minimum pathway leakage rate was 27.4 slm (58.1 SCFH). The evaluation determined that the MSIV LLRT excess leakage was reportable pursuant to 10CFR50.72(b)(2)(i) and 10CFR50.73(a)(2)(ii).

CAUSE OF EVENT

The as found leakage rate was attributed to the combined performance of the MSIVs; however, no definitive cause of this leakage rate could be determined. Like a prior similar event on Unit 1, valve stroking was the only form of "rework" necessary. It is theorized that inadequate seating prior to the "as found" testing was caused by closing the MSIVs at low power and low steam flow conditions.

REPORTABILITY/ANALYSIS

This event was determined to be reportable under 10CFR50.72(b)(2)(i), as a condition found while the reactor was shutdown, and 10CFR50.73(a)(2)(ii) in that MSL containment penetration leakage through both the inboard and outboard MSIVs was in excess of Technical Specification limits.

Based on an assessment of the safety consequences of previous similar events, performed by Nuclear Engineering, offsite and Control Room accident doses would remain within 10CFR100 and 10CFR50 limits. This assessment took into account the availability of MSIV Leakage Control System (MSIV-LCS, EIS Code: BD). The MSIV-LCS is designed to control and minimize the possible release of radioactive gases that could leak through the closed MSIVs following a Loss of Coolant Accident (LOCA). Additionally, this leakage is also bounded by PP&L's analysis that supported the approved deletion of the MSIV-LCS. The analysis results demonstrate that dose contributors from the approved MSIV leakage rate limit of 100 SCFH per steam line, not to exceed a total of 300 SCFH for all four main steam lines, along with the deletion of the Leakage Control System (LCS), result in an insignificant increase to the LOCA doses previously evaluated against the regulatory limits for the off-site doses and control room doses contained in 10CFR100 and 10CFR50, Appendix A, General Design Criterion (GDC) 19, respectively. Therefore, this event had no safety significance nor was there any risk to the health and safety of the public.

**LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION**

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

In accordance with the guidelines provided in NUREG-1022, Supplement 1, Item 14.1, the required submission date for this report was determined to be October 20, 1995.

CORRECTIVE ACTIONS

The "B" and "D" inboard and the "A" outboard MSL MSIVs were stroked and the MSIV LLRT reformed. The total "as left" containment MSL penetration minimum pathway leakage was measured at 13.8 sim (29.3 SCFH) and the maximum pathway "as left" leakage was measured at 31.2 sim (66.1 SCFH). No additional corrective actions were identified.

ADDITIONAL INFORMATION

Past Similar Events: LER 83-062-00, Docket No. 387/License No. NPF-14
 LER 83-064-00, Docket No. 387/License No. NPF-14
 LER 86-007-00, Docket No. 388/License No. NPF-22
 LER 89-010-01, Docket No. 388/License No. NPF-22
 LER 90-020-00, Docket No. 387/License No. NPF-14
 LER 92-005-00, Docket No. 387/License No. NPF-14
 LER 95-006-00, Docket No. 387/License No. NPF-14

Failed Component: MSIVs, HV-141F022B & D and HV-141F028A

Manufacturer: Atwood and Morrill Co., Inc.

Model: 21190-H

