

# PRIORITY 1

(ACCELERATED RIDS PROCESSING)

## REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 9510200159. DOC. DATE: 95/10/13 NOTARIZED: NO DOCKET #  
FACIL: 50-388 Susquehanna Steam Electric Station, Unit 2, Pennsylvania 05000388  
AUTH. NAME AUTHOR AFFILIATION  
KICHLINE, R.D. Pennsylvania Power & Light Co.  
STANLEY, H.G. Pennsylvania Power & Light Co.  
RECIP. NAME RECIPIENT AFFILIATION

SUBJECT: LER 95-010-00: on 950913, facility manually shutdown per TS  
Action 3.6.1.1 due to failed LLRT. Resilient seal for valve  
HV25724 replaced & four other drywell & suppression chamber  
purge supply valves resilient seals replaced. W/951013 ltr.

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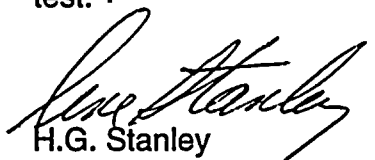
October 13, 1995

US Nuclear Regulatory Commission  
Document Control Desk  
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**SUSQUEHANNA STEAM ELECTRIC STATION**  
**LICENSEE EVENT REPORT 388/95-010-00**  
**PLAS- 648 FILE R41-2**

Docket No. 50-388  
License No. NPF-22

Attached is Licensee Event Report 388/95-010-00. This report is being made pursuant to 10CRF50.73(a)(2)(i) in that the Susquehanna Steam Electric Station - Unit 2 completed a reactor shutdown as required by the unit's Technical Specifications due to the failure of a containment isolation valve local leak rate test.

  
H.G. Stanley  
VP - Nuclear Operations

RDK/toc

cc: Mr. T.T. Martin  
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LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U7.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Susquehanna Steam Electric Station - Unit 2	DOCKET NUMBER(2) 0 5 0 0 0 3 8 8	PAGE (3) 1 OF 0 3
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TITLE (4)  
Manual Shutdown Per Technical Specifications Due To Failed LLRT

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0 9	1 3	9 5	9 5	0 1 0	0 0	1 0	1 3	9 5			0 5 0 0 0
											0 5 0 0 0

OPERATING MODE (9) 1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5 : (Check one or more of the following) (11)										
POWER LEVEL (10) 0 1 6	20.402(b)			20.405(c)			50.73(a)(2)(v)			73.71(b)	
	20.405(a)(1)(x)			50.36(c)(1)			50.73(a)(2)(v)			73.71(c)	
	20.405(a)(1)(y)			50.36(c)(2)			50.73(a)(2)(v)			OTHER (Specify in Abstract below and in Text, NRC Form 366A)	
	20.405(a)(1)(z)			X 50.73(a)(2)(f)			50.73(a)(2)(v)(A)				
	20.405(a)(1)(AA)			50.73(a)(2)(f)			50.73(1)(2)(v)(B)				
	20.405(a)(1)(AB)			50.73(a)(2)(g)			50.73(a)(2)(v)				

(LICENSEE CONTACT FOR THIS LER (12))

NAME Robert D. Kichline - Project Licensing Specialist - Licensing	TELEPHONE NUMBER AREA CODE 7 1 7 5 4 2 - 3 2 8 9
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC
X	B B	I I S V	P 3 4 0	Y					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (if yes, complete EXPECTED SUBMISSION DATE)	X NO	EXPECTED SUBMISSION DATE (15)	MONTH DAY YEAR
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ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

At 2332 hours on September 13, 1995, with Unit 2 in Condition 1 at 16% power the Unit was manually shutdown in conformance with Technical Specification ACTION 3.6.1.1. On September 13, 1995, the local leak rate test (LLRT) performed on the drywell and suppression chamber purge supply valves with resilient seals (containment isolation valves) failed the acceptance criteria. This testing is performed pursuant to Technical Specification 4.6.1.2 and Technical Specification 4.6.1.8.2 to assure compliance with 10CFR50 Appendix J criteria. Because the Technical Specification definition for PRIMARY CONTAINMENT INTEGRITY applied, Technical Specification ACTION 3.6.1.1 was applicable. The Unit 2 shutdown began the Unit's seventh refueling and inspection outage. Subsequent investigations determined that only the outboard suppression chamber purge supply valve (HV25724), failed the acceptance criteria. Since the inboard containment isolation valve LLRT values were acceptable, potential doses remained within 10CFR100 limits. The cause of the leakage from valve HV25724 was a mechanically induced failure. Corrective actions include: replacement of the resilient seal. Preventive maintenance improvements for these valves have been established that should reduce LLRT failures.

**LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION**

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)  Unit 2  Susquehanna Steam Electric Station	DOCKET NUMBER (2)  0   5   0   0   0   3   8   8	LER NUMBER (6)						PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER						
		9   5   —	0   1   0   —	0   0	2	OF	3			

TEXT (If more space is required, use additional NRC Form 366A's) (17)

**Description of the Event**

At 2332 hours on September 13, 1995, with Unit 2 in Condition 1 at 16% power the Unit was manually shutdown in accordance with Technical Specification ACTION 3.6.1.1. On September 13, 1995, the local leak rate test (LLRT) performed on the drywell and suppression chamber purge supply valves with resilient seals (containment isolation valves) (EIS Code: BB) failed the LLRT acceptance criteria as required by Technical Specifications 4.6.1.2 and 4.6.1.8.2. Operability as determined by these Technical Specifications assures compliance with 10CFR50 Appendix J criteria.

The Technical Specification definition for PRIMARY CONTAINMENT INTEGRITY requires that the primary containment leakage rates are within the limits of Technical Specification 3.6.1.2. Therefore, the failure to meet the criteria of Technical Specification 4.6.1.2 placed the Unit in Technical Specification ACTION 3.6.1.1. In conformance with Technical Specification ACTION 3.6.1.1, Unit 2 was shutdown. This shutdown began the Unit's seventh refueling and inspection outage.

**Cause of the Event**

Subsequent investigations determined that only the outboard suppression chamber purge supply valve (HV25724), failed the acceptance criteria. A mechanically induced failure mechanism (tearing of the resilient seal) was determined to have caused the LLRT failure. An evaluation of the removed resilient seal material is being conducted. The other drywell and suppression chamber purge supply containment isolation valves met the LLRT acceptance criteria.

**Reportability/Analysis**

This event was determined to be reportable pursuant to 10CFR50.73(a)(2)(i) in that Unit 2 completed a Unit shutdown as required by Technical Specification ACTION 3.6.1.1. Technical Specification ACTION 3.6.1.1 requires a unit shutdown if primary containment integrity cannot be demonstrated. Subsequent investigation, after the Unit was shut down, established that primary containment was maintained and that potential doses remained within 10CFR100 limits.

In accordance with the guidelines provided in NUREG-1022, Supplement 1, Item 14.1, the required submission date for this report was determined to be October 13, 1995.

**LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION**

FACILITY NAME (1)  Unit 2  Susquehanna Steam Electric Station	DOCKET NUMBER (2)  0   5   0   0   0   3   8   8	LER NUMBER (6)						PAGE (3)					
		YEAR		SEQUENTIAL NUMBER		REVISION NUMBER							
		9	5	—	0	1	0	—	0	0	3	OF	3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

**Corrective Actions**

The resilient seal for valve HV25724 was replaced. Additionally, the four other Unit 2 drywell and suppression chamber purge supply valves that have resilient seals have had their resilient seals replaced.

Preventive maintenance improvements for these valves were established in February 1994, and should assist in reducing some LLRT failures modes. This maintenance, which is based on engineering judgment, recommends that the resilient seals of these valves be replaced every seven and one half years. Although this determination is based on age degradation of the seals it should also assist in reducing mechanically induced failures.

Except for two Unit 1 valves, Henry Pratt valves in both Units that contain resilient seals have had the resilient seals replaced within the last seven and one half years. The two Unit 1 valves are scheduled to have their resilient seals replaced in the next Unit 1 refueling and inspection outage which is scheduled to begin in September 1996.

**Additional Information**

Past Similar Events: LER 87-014-00, Docket No. 387/License No. NPF-14

Failed Components: Suppression chamber purge supply containment isolation valve, HV25724

Manufacturer: Henry Pratt Co.

Model: 1200