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SUBJECT: Forwards "SSES Unit 2 Power Uprate Test Program Startup Rept," per Tech Specs 6.9.1.1,6.9.1.2 & 6.9.1.3.Power uprate test program successfully & safely completed on 940818.

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U.S. Nuclear Regulatory Commission
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SUSQUEHANNA STEAM ELECTRIC STATION
UNIT 2 POWER UPRATE TEST PROGRAM
PLA-4203 FILES A17-2/R41-2 **Docket No. 50-388**

Dear Sir:

This letter transmits the Susquehanna Steam Electric Station, Unit 2, "Power Uprate Test Program Startup Report" pursuant to Technical Specifications 6.9.1.1, 6.9.1.2, and 6.9.1.3.

Overall, the Power Uprate Test Program was successfully and safely completed on August 18, 1994, ahead of schedule with an overall increase in net electrical output slightly greater than anticipated. All Level 1 Acceptance Criteria (those which test design values) were satisfied. Only five Level 2 Acceptance Criteria exceptions were written (Level 2 Criteria are those which test system expectations): three concerning a single pre-power uprate existing condition, and two due to one set of anomalous GETARS data. Nine other exceptions were written, most of which were administrative in nature, none of which affected any Acceptance Criteria.

The SSES Unit 2 Power Uprate Test Program encompassed the scope of events commencing with the verification of the newly loaded reactor core and terminating with the completion of the final test program review. The program was conducted using the same considerations for tests and administrative controls used during the initial test program's startup test program. Formal tests, denoted as power uprate tests were conducted during this program. Testing and power escalation was sequenced in four distinct test plateaus:

JE24 1/1

Test Plateau	Test Condition
A/B	<90% uprated power level
C	95-96% uprated power level
D	97-98% uprated power level
E	99-100% uprated power level

The Startup Test Program demonstrated the unit can operate safely at uprated power conditions and that the unit's systems respond as expected. Increases in vibration of various components and systems associated with primary containment and a general increase in noise level near primary containment, however, were observed during testing. These phenomena occurred at higher core flows and were determined to be associated with the vane passing frequency of the reactor recirculation pumps. An administratively controlled limit on recirculation pump speed has been implemented while engineering work to resolve this issue continues. Our action plan was discussed with the NRC staff in a meeting held on October 6, 1994.

Should you have any questions regarding this submittal, they should be directed to Mr. W. W. Williams at (610) 774-5610.

Very truly yours,

R.G. Byram

Attachment

cc: NRC Region I
Ms. M.Banerjee, NRC Sr. Resident Inspector - SSES
Mr. C. Poslusny, Jr., NRC Sr. Project Manager - OWFN