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Susquehanna
Training Center

August 11, 1994

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Susquehanna Training Center
Facility Comments
PLES 560 File A14-13D

Enclosed you will find five questions for your review. In accordance with ES 402, Attachment 3, paragraphs 4, 5, and 6, we have included the question and exam key answer, the facility comment, supporting reference, and facility recommendation.

We hope these comments will be considered during the grading process.

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NTG File
Nuc Records - Site

Enclosures

BRS/8-1-i

HGS/BRS/mjp

QUESTION: 002 (1.00)

Unit 1 was operating at 100% power when a large break LOCA occurred. Reactor water level decreased to approximately -170 inches and has not yet been restored five minutes after the event occurred. The operator performing ON-159-002, "Containment Isolation," reports that the drywell nitrogen makeup inboard and outboard isolation valves are stuck open. SELECT the proper emergency classification.

a. Unusual event

b. Alert

c. Site emergency

d. General emergency

NRC Answer Key:

c

Facility Comment:

The classification of this event would be affected by the assumption made about the location of the large break LOCA (inside drywell or outside drywell). The stated conditions qualify as entry conditions for a general emergency in accordance with EP-PS-100, EAL 12.4b.

Facility Recommendation:

Accept either answer c or d

QUESTION: 015 (1.00)

Which of the following describes the conditions which will allow opening the MSIVs with condenser vacuum at 2.5 inches Hg if the low vacuum bypass switches are in BYPASS?

a. Mode switch in RUN
Main turbine stop valves closed
Main turbine control valves open

b. Mode switch in RUN
Main turbine stop valves closed
Main turbine control valves closed

c. Mode switch in Startup
Main turbine control valves closed
Main turbine stop valves open

d. Mode switch in Startup
Main turbine stop valves closed
Main turbine control valves open

NRC Answer Key:

c

Facility Comment:

The correct answer is d. The reference SY017 H-2, page 13, is a typo. It should read stop valve < 90% open.

Facility Recommendation:

Change correct answer to d.

QUESTION: 035 (1.00)

Unit 1 is operating at 100% power. Diesel generator 'A' is out of service for extensive repair and has been replaced with diesel generator 'E'. The following equipment is out of service:

480 VAC load center 1B280
480 VAC load center 1B210
480 VAC motor control center OB516
480 VAC motor control center 1B227

SELECT the statement that is in accordance with technical specifications from those listed below:

a. Re-energize the equipment within 24 hours or be in at least hot shutdown within the next 12 hours and cold shutdown within the following 24 hours.

b. Re-energize the equipment within 8 hours or be in at least hot shutdown within the next 12 hours and cold shutdown within the following 24 hours.

c. Re-energize the equipment within 2 hours or be in at least hot shutdown within the next 12 hours and cold shutdown within the following 24 hours.

d. Re-energize the equipment or be in at least hot shutdown within the next 6 hours and in cold shutdown within the following 24 hours.

NRC Answer Key:

d

Facility Comment:

No correct answer is presented. Technical specification 3.8.3.1 action a addresses a loss of either a division 1 (1B210) or division 2 (1B227) AC distribution system load group. With both a division 1 and division 2 AC distribution system load group out of service technical specification 3.0.3 is applicable. No answer choice is as conservative as T.S. 3.0.3 action.

Facility Recommendation:

Delete the question.

QUESTION: 078 (1.00)

A transient on unit 1 resulted in a loss of coolant. The following plant conditions exist:

Reactor scrammed
All rods at 00
Div 1 low pressure ECCS has initiation signal
Div 2 low pressure ECCS has no initiation signal
HPCI and RCIC are injecting
RPV level is -128 inches on wide range
RPV pressure is 650 psig
Suppression pool temperature is 109 degrees F
Drywell temperature is 180 degrees F
Drywell pressure is 6.2 psig
Suppression pool level is 23 feet
No secondary containment problems exist

Which of the following statements is correct?

a. Div 2 low pressure ECCS failed to start on high drywell pressure, the operator should start core spray and LPCI

b. Wide range instrument may be used to determine RPV level

c. RPV level cannot be determined and EO-100-114, RPV Flooding, should be entered

d. Adequate core cooling is assured and one loop of LPCI should be placed in drywell spray

NRC Answer Key:

b

Facility Comment:

Insufficient data in the question stem prevents analysis of the answer choices c and d. In accordance with EOP caution 1, wide range reactor water level is not usable below -125 inches unless the instrument range is determined usable below this value in accordance with ON-145-004, Attachment C. This would require memorization of level correction curves.

Facility Recommendation:

Delete the question.

QUESTION: 091 (1.00)

The reactor automatically scrams due to high level in the scram discharge volume. How soon is the NRC Operations Center required to be notified?

a. Within 1 hour

b. Within 4 hours

c. Within 24 hours

d. Within 36 hours

NRC Answer Key:

a

Facility Comment:

A reactor scram (ESF actuation) is reportable per NDAP-QA-0724, Attachment G, Table B as a 4 hour ENS notification.

Facility Recommendation:

Change correct answer to b.

ATTACHMENT 3

Resolution of Facility Comments

1. Question 2 Comment accepted. Answer c or d accepted as correct

2. Question 15 Comment accepted. Correct answer changed to d

3. Question 35 Comment accepted. Question deleted

4. Question 78 Comment accepted. Question deleted

5. Question 91 Comment accepted. Correct answer changed to b

ATTACHMENT 4

SIMULATOR FACILITY REPORT

Facility Licensee: Susquehanna Steam Electric Station

Facility Docket No: 50-387/388

Operating Tests Administered August 9, 1994

This form is to used only to report observations. These observations do not constitute audit of inspection findings and are not, without further verification and review, indicative of noncompliance with 10 CFR 55.45(b). These observations do not affect NRC certification or approval of the simulation facility other than to provide information that may be used in future evaluations. No licensee action is required in response to these observations.

While conducting the simulator portion of the operating tests, the following items were observed (if none, so state):

ITEM	DESCRIPTION
None	