


REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 9408040408 DOC. DATE: 94/07/29 NOTARIZED: YES DOCKET #
 FACIL: 50-387 Susquehanna Steam Electric Station, Unit 1, Pennsylv 05000387
 50-388 Susquehanna Steam Electric Station, Unit 2, Pennsylv 05000388
 AUTH. NAME AUTHOR AFFILIATION
 BYRAM, R.G. Pennsylvania Power & Light Co.
 RECIPIENT AFFILIATION
 MILLER, C.L. Project Directorate I-2

REV. 7/15/95 

SUBJECT: Forwards description of Rev 47 changes to FSAR for SSES. Most current rev of design drawings incorporated in Rev 47 to FSAR. Amended pages being provided to controlled distribution under separate cover.

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Pennsylvania Power & Light Company

Two North Ninth Street • Allentown, PA 18101-1179 • 610/774-5151

Robert G. Byram
Senior Vice President—Nuclear
610/774-7502
Fax: 610/774-5019

JUL 29 1994

Director of Nuclear Reactor Regulation
Attention: Mr. C. L. Miller, Project Director
Project Directorate I-2
Division of Reactor Projects
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

**SUSQUEHANNA STEAM ELECTRIC STATION
FINAL SAFETY ANALYSIS REPORT
REVISION 47
PLA-4181**

Docket Nos. 50-387
and 50-388

FILES R41-2, A17-1

Dear Mr. Miller:

Pursuant to 10CFR50.71, Pennsylvania Power & Light (PP&L) Company is submitting Revision 47 to the Susquehanna SES Final Safety Analysis Report (FSAR). The changes contained in Revision 47 were processed in accordance with 10CFR50.59, were required by the NRC, were subjected to a No Significant Hazards Evaluation, or are administrative in nature. The amended pages are being provided to the controlled distribution under separate cover.

Attachment 1 provides a description of each change. The most current revision of design drawings have also been incorporated as appropriate however, a description of the actual change(s) to each drawing is not provided.

In addition, Susquehanna Steam Electric Station's (SSES) future FSAR updates will exercise the option provided by NRC's amendment of 10CFR50.71(e) resulting in updates to the FSAR being provided no later than 6 months after each refueling outage. This revised update interval is planned to be tied to the SSES Unit 2 refueling outage schedule, resulting in the next FSAR Amendment being submitted no later than March, 1996.

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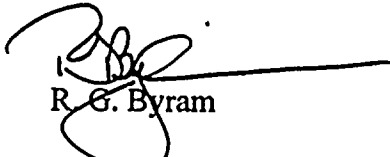
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Inclusion*

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A final administrative enhancement being implemented within PP&L, via the guidance of NRC Generic Letter 88-18, is the replacement of the Silver Halide microfilm with an Optical Disk Storage System for the archival storage of records. Utilization of this storage medium will allow PP&L to enhance record capture capability, reduce storage needed to accommodate bulk microfilm and hardcopy, and ensure quick retrievability of stored records.

If you have any questions regarding any of the above, please contact Mr. A.K. Maron at (610) 774-7852.

Very truly yours,



R. G. Byram

Attachment

cc: NRC Document Control Desk (original)
NRC Region I
Mr. G. S. Barber, NRC Sr. Resident Inspector
Mr. C. Poslusny, Jr., NRC Sr. Project Manager

BEFORE THE
UNITED STATES NUCLEAR REGULATORY COMMISSION

In the Matter of :

PENNSYLVANIA POWER & :
LIGHT COMPANY

Docket No. 50-387
and 50-388

REVISION NO. 47

FINAL SAFETY ANALYSIS REPORT
FOR THE
SUSQUEHANNA STEAM ELECTRIC STATION

UNITS NO. 1 AND NO. 2

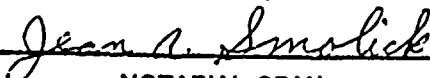
Pennsylvania Power & Light Company, hereby files Revision 47 to its Final Safety Analysis Report dated July 31, 1978.

PENNSYLVANIA POWER & LIGHT COMPANY
BY:



R. G. Byram
Sr. Vice President - Nuclear

Sworn to and subscribed before me
this 29th of July, 1994.



NOTARIAL SEAL
Jean A. Smolick, Notary Public
Allentown, Lehigh County
My Commission Expires May 14, 1996



SECTION	DESCRIPTION
1.2.2.3.1, 1.6, 3.1.2.3.7, 3.9.1.4.10, 3.9.2.2a.2.5, 3.9.7, 4.1, 4.2	Subsections revised to incorporated previously approved licensing and design bases to more accurately reflect current plant configuration.
Table 1.7-1	Updated to reflect current drawing revision levels.
Table 1.8-1	Revised definition to maintain consistency with 10CFR50.62 definition.
Tables 3.9-4 through 3.9-33, Figures 3.9-1 through 3.9-11	Editorial changes to remove block Tables and Figures.
3.1.2.1.5, 8.1.6.1, 8.2.1.1, 8.3.1.11.1	Subsection revised to enhance the complete listing of the systems requiring compliance to the intent of GDC 5, "Sharing of Structures, Systems, and Components."
Table 3.2-1, 9.2.5	Table and Subsection revised to reflect replacement tube bundles for the jacket water heat exchangers.
3.6, Table 3.6-10, Figure 3.6-5	Revised pipe stress, cumulative factor, and usage pipe break stress limit as a result of Snubber Elimination on the Core Spray Line inside containment.
3.11.1	Editorial change to enhance the description of the NSSS and non - NSSS equipment.
3.11.2b.2	Subsection revised to include the listing of Regulatory Guides 1.97 and 1.131 relating to environmental qualification of non-NSSS safety-related equipment.
3.13.1, 8.1.6.1d	Subsection revised to more accurately address electric power system compliance to Regulatory Guide 1.22.
3.13.1, 8.1.6.1	Subsections revised to reflect cross reference and compliance with Reg. Guide 1.62.
5.4.7.2.2, 9.2.5.1, 9.2.5.2, Tables 9.2-2, 9.2-3, 9.2-4, 9.2-5, Figure 9.2-5b	Subsections, Tables and Figure revised to reflect the replacement of RHR Pump Seal with a new "HOT" seal which does not require an external cooling medium.
5.4.8.2, 9.2.10.1, Table 5.4-2, Figures 5.4-16, 5.4-17b, 7.7-11, 9.2-2.	Subsections, Table, and Figures revised to incorporate the new sealless RWCU pumps (100% capacity).
7.4.1.4.2.3	Subsection revised to reflect current operation of RHR Shutdown Cooling Mode from the Unit 2 Remote Shutdown Panel, and correct description of the method of flushing the Shutdown Cooling Piping.

SECTION	DESCRIPTION
Table 7.5-3	Table revised to clarify information and change the Suppression Pool Level indication accuracy from ± 2 to ± 2 .
7.7.1.7.3.1.1, 7.7.1.7.3.1.4, 7.7.1.7.5.1	Subsections updated to reflect the new computer installed and to delete reference to original NSS calculation methodology.
8.2.1.1, 8.2.1.3.6, 8.2.1.3.1	Subsections revised to reflect editorial figure number changes.
8.1.6.1	Subsection revised to reflect the replacement of the word Frequency with Voltage, to provide the existing/proper design basis of the Diesel Generator & Auxiliaries system.
8.1.6.1v)	Subsection revised to adopt Position 2 of Reg Guide 1.106 to ensure the affected valves perform their intended safety function during accident events while meeting the requirements of the Reg Guide.
Table 8.1-2	Table revised to add the affiliated circuit, Non-Class 1E load description, Class 1E power supply and identify method of isolation.
8.3.1.3.10, Table 8.3-1	Subsection and Table revised to provide a clearer interpretation concerning ESF load sequencing.
8.3.1.4.1	Revised discussion on circuit redundancy and circuit failure.
8.3.2.2.1a, 8.3.2.2.1c	Subsections revised to provide a more accurate depiction of the DC subsystem design.
Table 8.3-1	Table revised to update information associated with the In Line Process Instrumentation Upgrade Project, along with deleting the note identifying maximum MOV stroke time.
9.1.4.2.5.7	Subsection revised to reflect the replacement of wire rope and hook box with Kevlar slings.
9.1.4.2.10.2.2, 9.1.4.2.10.2.4	Subsections revised to reflect current sequence for reactor shutdown and reactor well shield plug removal.
9.2.5.1, 9.2.5.2, Table 9.2-3, 9.2-4, 9.2-5	Subsection revised to eliminate the seal coolers and cooling water to the RHR pump seals.
9.2.5.2, 9.2.6.2	Subsection revised to clarify design for RHRSW/ESW systems.

SECTION	DESCRIPTION
9.3.3.2.3	Subsection revised to describe manually draining of the condenser area transfer sump.
9.4.3.5	Subsection revised to reflect the replacement of the switch function to a flow switch which will eliminate need for seasonal bypasses.
9.4.4.5	Subsection revised to reflect the elimination of the alarm for the turbine building high and low building pressures.
Table 9.4-1, 9.4-7	Tables revised to replace the design limit value with the process setpoint
9.5.2.2.3	Subsection revised to add wording to permit use of alternate communication equipment to maintain direct communication between the control room and refueling floor personnel
Table 9A-1	Correction of typographical error, duration of accident is defined as 30 days, not 80.
11.3.1.2, 11.3.2.1.1, 11.3.2.2.8, 11.3.3, 15B, Figures 11.3-1 through 11.3-5	Figures renumbered to eliminate blank pages; renumbered Figures reflected in text.
11.2.2.4, Table 11.2-3	Subsection and Table revised to reflect the replacement of the Laundry Drain Filters, with a triplex filter.
12.3.2.2.5, 12.3.4.1.3, Figures 12.3-28 through 12.3-30	Revised to add the LLRW Holding Facility Interim Radiation Zoning and Area Radiation Monitor Locations.
12.3, 12.5	Sections revised to incorporate revision to 10CFR20, and the implementation of new radiation dose limits.
12.5.3.7.1.1	Subsection revised to align management review of Health Physics Training Program in line with the Health Physics evaluation program.
Table 12.3-7	Table revised to reflect the relocation of ARM Channel 11 to an area outside room I-508 adjacent to the new Reactor Building Sample Station.
13.1.2.1, 13.1.2.2.1, 13.1.3.1.1, Tables 13.1-1, 13.1-2, Figures 13.1-1, 13.1-3a, 3c, 3d, 13.1-4, 13.1-5	Subsections, Tables, and Figures revised to reflect current organizational structure.

SECTION	DESCRIPTION
15.4.1.2	Subsection revised to clarify description of GE's generic analysis of the continuous withdrawal of an out-of-sequence control rod during startup.
15A.6.4.3	Editorial change to reflect correct Figure reference.
17.2, 17.2.1.1, 17.2.1.1.3, 17.2.2, Tables 17.2-1, 17.2-3	Subsections and Tables revised to reflect changes in organization, including clarification and title changes.
18.1.3.3, 18.1.4.3, 18.1.13.3, 18.2.2.3, 18.2.8.3, 18.2.9.3, 18.2.10.3, 18.2.26.3.	Subsections revised to reflect correct procedure number references.
Q 021.74, Q 331.1	Editorial correction of typographical errors.
Q 040.3	Response to question revised to reflect the addition of Rockbestos cables used for multi-pin quick disconnect connectors for MOVs.
Q 211.194	Editorial changes to reflect correct referenced subsections.
Q 421	Clarify entire question response to reflect historical data, identifying all current changes to the QA Program being reflected in Chapter 17.2.





Pennsylvania Power & Light Company

Two North Ninth Street • Allentown, PA 18101-1179 • 610/774-5151

Robert G. Byram
Senior Vice President—Nuclear
610/774-7502
Fax: 610/774-5019

JUL 29 1994

Director of Nuclear Reactor Regulation
Attention: Mr. C. L. Miller, Project Director
Project Directorate I-2
Division of Reactor Projects
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

**SUSQUEHANNA STEAM ELECTRIC STATION
FINAL SAFETY ANALYSIS REPORT
REVISION 47
PLA-4181**

FILES R41-2, A17-1

**Docket Nos. 50-387
and 50-388**

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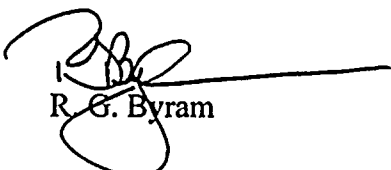


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Very truly yours,


R. G. Byram
Attachment

cc: NRC Document Control Desk (original)
NRC Region I
Mr. G. S. Barber, NRC Sr. Resident Inspector
Mr. C. Poslusny, Jr., NRC Sr. Project Manager



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PENNSYLVANIA POWER & LIGHT COMPANY
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R. G. Byram
Sr. Vice President - Nuclear

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this 29th of July, 1994.

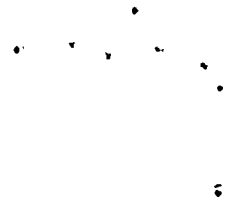
Jean A. Smolick

NOTARIAL SEAL
Jean A. Smolick, Notary Public
Allentown, Lehigh County
My Commission Expires May 14, 1996

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12.3, 12.5	Sections revised to incorporate revision to 10CFR20, and the implementation of new radiation dose limits.
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15A.6.4.3	Editorial change to reflect correct Figure reference.
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18.1.3.3, 18.1.4.3, 18.1.13.3, 18.2.2.3, 18.2.8.3, 18.2.9.3, 18.2.10.3, 18.2.26.3.	Subsections revised to reflect correct procedure number references.
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Q 211.194	Editorial changes to reflect correct referenced subsections.
Q 421	Clarify entire question response to reflect historical data, identifying all current changes to the QA Program being reflected in Chapter 17.2.

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ACCESSION NBR: 9408040408 DOC. DATE: 94/07/29 NOTARIZED: YES DOCKET #
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 AUTH. NAME AUTHOR AFFILIATION
 BYRAM, R.G. Pennsylvania Power & Light Co.
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 MILLER, C.L. Project Directorate I-2

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NOTE TO ALL "RIDS" RECIPIENTS:

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Pennsylvania Power & Light Company

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Robert G. Byram
Senior Vice President—Nuclear
610/774-7502
Fax: 610/774-5019

JUL 29 1994

Director of Nuclear Reactor Regulation
Attention: Mr. C. L. Miller, Project Director
Project Directorate I-2
Division of Reactor Projects
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

**SUSQUEHANNA STEAM ELECTRIC STATION
FINAL SAFETY ANALYSIS REPORT
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A final administrative enhancement being implemented within PP&L, via the guidance of NRC Generic Letter 88-18, is the replacement of the Silver Halide microfilm with an Optical Disk Storage System for the archival storage of records. Utilization of this storage medium will allow PP&L to enhance record capture capability, reduce storage needed to accommodate bulk microfilm and hardcopy, and ensure quick retrievability of stored records.

If you have any questions regarding any of the above, please contact Mr. A.K. Maron at (610) 774-7852.

Very truly yours,



R. G. Byram

Attachment

cc: NRC Document Control Desk (original)
NRC Region I
Mr. G. S. Barber, NRC Sr. Resident Inspector
Mr. C. Poslusny, Jr., NRC Sr. Project Manager


BEFORE THE
UNITED STATES NUCLEAR REGULATORY COMMISSION

In the Matter of :
PENNSYLVANIA POWER & : Docket No. 50-387
LIGHT COMPANY : and 50-388

REVISION NO. 47
FINAL SAFETY ANALYSIS REPORT
FOR THE
SUSQUEHANNA STEAM ELECTRIC STATION
UNITS NO. 1 AND NO. 2

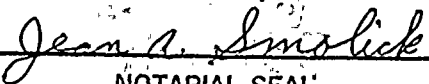
Pennsylvania Power & Light Company, hereby files Revision 47 to its Final Safety Analysis Report dated July 31, 1978.

PENNSYLVANIA POWER & LIGHT COMPANY
BY:



R. G. Byram
Sr. Vice President - Nuclear

Sworn to and subscribed before me
this 29th of July, 1994.


NOTARIAL SEAL
Jean A. Smolick, Notary Public
Allentown, Lehigh County
My Commission Expires May 14, 1996



A



SECTION	DESCRIPTION
1.2.2.3.1, 1.6, 3.1.2.3.7, 3.9.1.4.10, 3.9.2.2a.2.5, 3.9.7, 4.1, 4.2	Subsections revised to incorporated previously approved licensing and design bases to more accurately reflect current plant configuration.
Table 1.7-1	Updated to reflect current drawing revision levels.
Table 1.8-1	Revised definition to maintain consistency with 10CFR50.62 definition.
Tables 3.9-4 through 3.9-33, Figures 3.9-1 through 3.9-11	Editorial changes to remove block Tables and Figures.
3.1.2.1.5, 8.1.6.1, 8.2.1.1, 8.3.1.11.1	Subsection revised to enhance the complete listing of the systems requiring compliance to the intent of GDC 5, "Sharing of Structures, Systems, and Components."
Table 3.2-1, 9.2.5	Table and Subsection revised to reflect replacement tube bundles for the jacket water heat exchangers.
3.6, Table 3.6-10, Figure 3.6-5	Revised pipe stress, cumulative factor, and usage pipe break stress limit as a result of Snubber Elimination on the Core Spray Line inside containment.
3.11.1	Editorial change to enhance the description of the NSSS and non - NSSS equipment.
3.11.2b.2	Subsection revised to include the listing of Regulatory Guides 1.97 and 1.131 relating to environmental qualification of non-NSSS safety-related equipment.
3.13.1, 8.1.6.1d	Subsection revised to more accurately address electric power system compliance to Regulatory Guide 1.22.
3.13.1, 8.1.6.1	Subsections revised to reflect cross reference and compliance with Reg. Guide 1.62.
5.4.7.2.2, 9.2.5.1, 9.2.5.2, Tables 9.2-2, 9.2-3, 9.2-4, 9.2-5, Figure 9.2-5b	Subsections, Tables and Figure revised to reflect the replacement of RHR Pump Seal with a new "HOT" seal which does not require an external cooling medium.
5.4.8.2, 9.2.10.1, Table 5.4-2, Figures 5.4-16, 5.4-17b, 7.7-11, 9.2-2.	Subsections, Table, and Figures revised to incorporate the new sealless RWCU pumps (100% capacity).
7.4.1.4.2.3	Subsection revised to reflect current operation of RHR Shutdown Cooling Mode from the Unit 2 Remote Shutdown Panel, and correct description of the method of flushing the Shutdown Cooling Piping.

SECTION	DESCRIPTION
Table 7.5-3	Table revised to clarify information and change the Suppression Pool Level indication accuracy from ± 2 to ± 2 .
7.7.1.7.3.1.1, 7.7.1.7.3.1.4, 7.7.1.7.5.1	Subsections updated to reflect the new computer installed and to delete reference to original NSS calculation methodology.
8.2.1.1, 8.2.1.3.6, 8.2.1.3.1	Subsections revised to reflect editorial figure number changes.
8.1.6.1	Subsection revised to reflect the replacement of the word Frequency with Voltage, to provide the existing/proper design basis of the Diesel Generator & Auxiliaries system.
8.1.6.1v)	Subsection revised to adopt Position 2 of Reg Guide 1.106 to ensure the affected valves perform their intended safety function during accident events while meeting the requirements of the Reg Guide.
Table 8.1-2	Table revised to add the affiliated circuit, Non-Class 1E load description, Class 1E power supply and identify method of isolation.
8.3.1.3.10, Table 8.3-1	Subsection and Table revised to provide a clearer interpretation concerning ESF load sequencing.
8.3.1.4.1	Revised discussion on circuit redundancy and circuit failure.
8.3.2.2.1a, 8.3.2.2.1c	Subsections revised to provide a more accurate depiction of the DC subsystem design.
Table 8.3-1	Table revised to update information associated with the In Line Process Instrumentation Upgrade Project, along with deleting the note identifying maximum MOV stroke time.
9.1.4.2.5.7	Subsection revised to reflect the replacement of wire rope and hook box with Kevlar slings.
9.1.4.2.10.2.2, 9.1.4.2.10.2.4	Subsections revised to reflect current sequence for reactor shutdown and reactor well shield plug removal.
9.2.5.1, 9.2.5.2, Table 9.2-3, 9.2-4, 9.2-5	Subsection revised to eliminate the seal coolers and cooling water to the RHR pump seals.
9.2.5.2, 9.2.6.2	Subsection revised to clarify design for RHRSW/ESW systems.

<i>SECTION</i>	<i>DESCRIPTION</i>
9.3.3.2.3	Subsection revised to describe manually draining of the condenser area transfer sump.
9.4.3.5	Subsection revised to reflect the replacement of the switch function to a flow switch which will eliminate need for seasonal bypasses.
9.4.4.5	Subsection revised to reflect the elimination of the alarm for the turbine building high and low building pressures.
Table 9.4-1, 9.4-7	Tables revised to replace the design limit value with the process setpoint
9.5.2.2.3	Subsection revised to add wording to permit use of alternate communication equipment to maintain direct communication between the control room and refueling floor personnel
Table 9A-1	Correction of typographical error, duration of accident is defined as 30 days, not 80.
11.3.1.2, 11.3.2.1.1, 11.3.2.2.8, 11.3.3, 15B, Figures 11.3-1 through 11.3-5	Figures renumbered to eliminate blank pages; renumbered Figures reflected in text.
11.2.2.4, Table 11.2-3	Subsection and Table revised to reflect the replacement of the Laundry Drain Filters, with a triplex filter.
12.3.2.2.5, 12.3.4.1.3, Figures 12.3-28 through 12.3-30	Revised to add the LLRW Holding Facility Interim Radiation Zoning and Area Radiation Monitor Locations.
12.3, 12.5	Sections revised to incorporate revision to 10CFR20, and the implementation of new radiation dose limits.
12.5.3.7.1.1	Subsection revised to align management review of Health Physics Training Program in line with the Health Physics evaluation program.
Table 12.3-7	Table revised to reflect the relocation of ARM Channel 11 to an area outside room I-508 adjacent to the new Reactor Building Sample Station.
13.1.2.1, 13.1.2.2.1, 13.1.3.1.1, Tables 13.1-1, 13.1-2, Figures 13.1-1, 13.1-3a, 3c, 3d, 13.1-4, 13.1-5	Subsections, Tables, and Figures revised to reflect current organizational structure.

SECTION	DESCRIPTION
15.4.1.2	Subsection revised to clarify description of GE's generic analysis of the continuous withdrawal of an out-of-sequence control rod during startup.
15A.6.4.3	Editorial change to reflect correct Figure reference.
17.2, 17.2.1.1, 17.2.1.1.3, 17.2.2, Tables 17.2-1, 17.2-3	Subsections and Tables revised to reflect changes in organization, including clarification and title changes.
18.1.3.3, 18.1.4.3, 18.1.13.3, 18.2.2.3, 18.2.8.3, 18.2.9.3, 18.2.10.3, 18.2.26.3.	Subsections revised to reflect correct procedure number references.
Q 021.74, Q 331.1	Editorial correction of typographical errors.
Q 040.3	Response to question revised to reflect the addition of Rockbestos cables used for multi-pin quick disconnect connectors for MOVs.
Q 211.194	Editorial changes to reflect correct referenced subsections.
Q 421	Clarify entire question response to reflect historical data, identifying all current changes to the QA Program being reflected in Chapter 17.2.

