



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

May 25, 1994

Docket Nos. 50-387
and 50-388

Mr. Robert G. Byram
Senior Vice President-Nuclear
Pennsylvania Power and Light
Company
2 North Ninth Street
Allentown, Pennsylvania 18101

Dear Mr. Byram:

SUBJECT: RESPONSE TO NRC BULLETIN 93-03, "RESOLUTION OF ISSUES RELATED TO REACTOR VESSEL WATER LEVEL INSTRUMENTATION IN BWRs" (TAC NOS. M86912 AND M86913)

We acknowledge receipt of your responses, dated July 29, 1993 and February 17, 1994 to NRC Bulletin (NRCB) 93-03. The NRCB was issued May 28, 1993, to all holders of operating licenses or construction permits for boiling water reactors (BWRs) with the exception of Millstone II and Big Rock Point. The NRCB requested licensees to take short-term compensatory measures within 15 days of the NRCB to ensure that operators were able to identify and mitigate to implement hardware modifications at the first cold shutdown after July 30, 1993, to ensure the level instrumentation system design would be of high functional reliability for long-term operation.

Your July 29, 1993 response stated that you implemented appropriate short-term compensatory measures. These were found to be acceptable to the staff. Subsequently you installed backfill systems to purge the non-condensable gases from the reference legs on each Susquehanna Unit. Specifically, the modifications were installed in March 1993 for Unit 1 and in February 1994 for Unit 2. Your letter of February 17, 1994 provided the report which confirmed completion and implementation of the required modifications. The dates of installation were consistent with NRCB 93-03. The staff understands that the modifications did not involve an unreviewed safety question and were implemented under 10 CFR 50.59.

Based on the information provided in your February 17, 1994 letter, we consider your response to NRCB 93-03 acceptable. Please retain records related to your actions taken in response to this NRCB for possible future inspection by the NRC staff.

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In a letter dated January 19, 1994, PP&L described a passive condensing chamber vent modification which was being developed and tested for installation on the Susquehanna units. The staff understands that the modification to install this passive alternative vessel level measurement system is being implemented under 10 CFR 50.59. An information meeting is being set up for PP&L to present design and testing information for this system to the staff in June 1994. This modification is being installed on Unit 2 during the current outage and will be installed on Unit 1 at the next outage.

The NRC staff issued NRC Information Notice (IN) 93-89, "Potential Problems With BWR Level Instrumentation Backfill Modifications." The staff is still reviewing the concerns identified in this IN, and any possible actions will be addressed separately from this TAC.

Sincerely,

/S/

Chester Poslusny, Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

cc: See next page

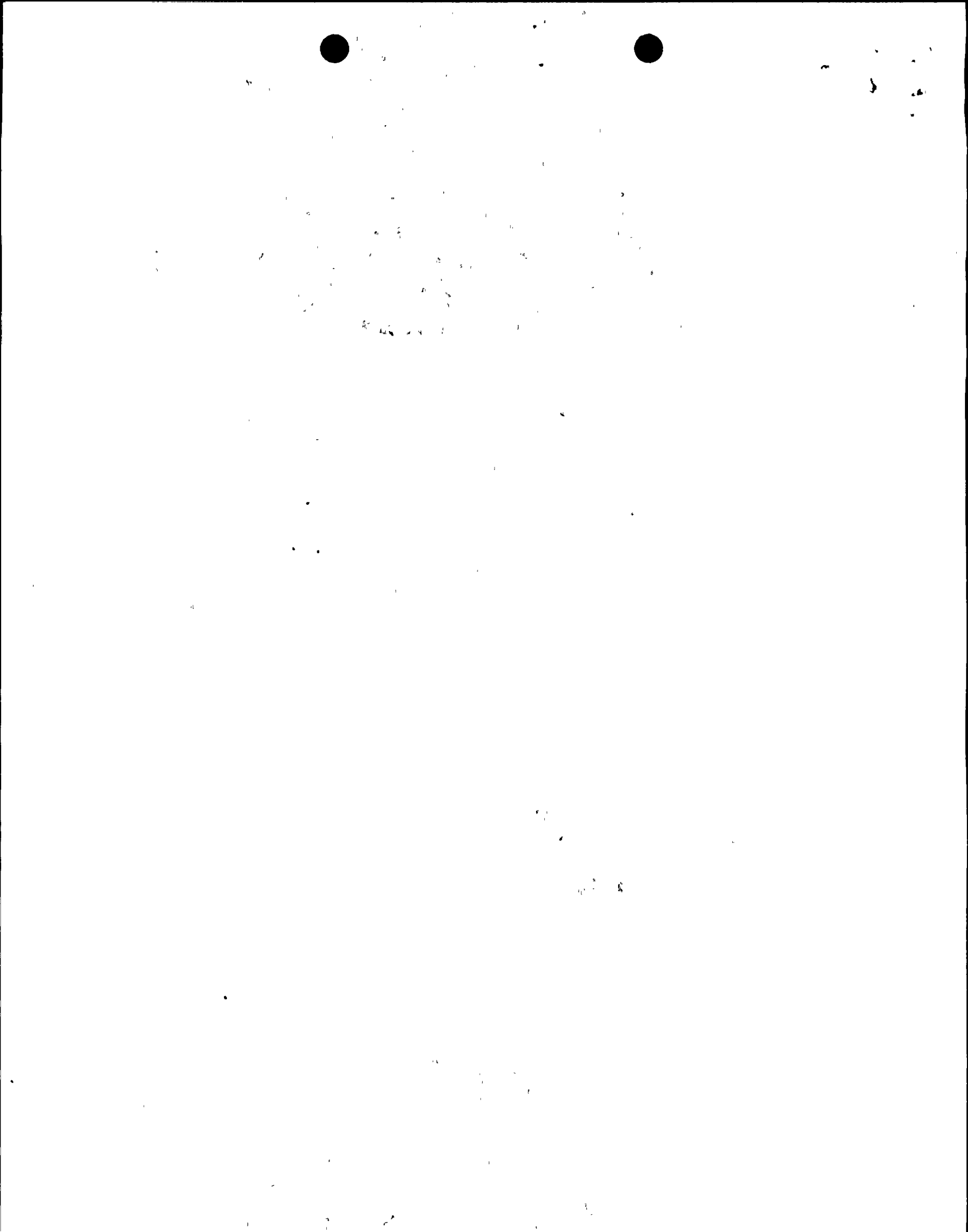
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Mr. Robert G. Byram

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Chester Poslusny, Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

cc: See next page

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