

# ACCELERATED DISTRIBUTION DEMONSTRATION SYSTEM

## REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 9312160364      DOC. DATE: 93/12/06      NOTARIZED: NO      DOCKET #  
 FACIL: 50-388 Susquehanna Steam Electric Station, Unit 2, Pennsylvania      05000388  
 AUTH. NAME      AUTHOR AFFILIATION  
 METER, J. J.      Pennsylvania Power & Light Co.  
 STANLEY, H. G.      Pennsylvania Power & Light Co.  
 RECIP. NAME      RECIPIENT AFFILIATION

SUBJECT: LER 93-006-00: on 931106, main steam line high flow switch lower assembly replaced due to erratic indication. Caused by mechanical binding of components within flow switch lower switch assembly due to wear. Switch replaced. W/931206 ltr.

DISTRIBUTION CODE: IE22T      COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 4  
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Pennsylvania Power & Light Company

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December 16, 1993

U.S. Nuclear Regulatory Commission  
Document Control Desk  
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SUSQUEHANNA STEAM ELECTRIC STATION  
LICENSEE EVENT REPORT 93-006-00  
PLAS - 581 FILE R41-2

Docket No. 50-388  
License No. NPF-22

Attached is Licensee Event Report 93-006-00. This report is being made pursuant to 10CFR50.73(a)(2)(i)(B), in that Susquehanna Unit 2 was in a condition prohibited by the Technical Specifications when Technical Specification 3.0.3 was entered to perform Operability testing of a Main Steam Line high flow switch after replacing the flow switch lower switch assembly. This condition was cleared once the lower switch assembly was replaced and restored to operable status.

  
H.G. Stanley  
VP - Nuclear Operations

JJM/mjm

cc: Mr. T. T. Martin  
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LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 60.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Susquehanna Steam Electric Station - Unit 2		DOCKET NUMBER (2) 0 5 0 0 0 3 8 8	PAGE (3) 1 OF 0 3
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TITLE (4)  
Main Steam High Flow Switch Testing Required Entry Into Technical Specification 3.0.3

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)															
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)													
1	1	0	6	9	3	9	3	0	0	0	6	0	0	1	2	0	6	9	3	0	5	0	0	0

OPERATING MODE (9) 1

POWER LEVEL (10) 1 0 0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.405(c)	<input type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)
<input type="checkbox"/> 20.405(a)(1)(i)	<input type="checkbox"/> 50.36(c)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(c)
<input type="checkbox"/> 20.405(a)(1)(ii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
<input type="checkbox"/> 20.405(a)(1)(iii)	<input checked="" type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(vii)(A)	
<input type="checkbox"/> 20.405(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(vii)(B)	
<input type="checkbox"/> 20.405(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(viii)	

LICENSEE CONTACT FOR THIS LER (12)

NAME Joseph J. Meter - Power Production Egnieer	TELEPHONE NUMBER AREA CODE 7 1 7 5 4 2 - 1 8 7 3
--	---

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
X	J   M	8   0	B   0   8   0	N					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)  NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On November 6, 1993, at 1323 hours with Unit 2 in Condition 1 at 100% power, the 'A' Main Steam Line high flow switch (FIS-B21-2N006B) lower switch assembly was replaced due to erratic indication. During replacement of the flow switch lower switch assembly, the trip function of the switch was placed in the tripped condition. At 1437 hours, in order to perform the subsequent functional and operability testing, the affected trip function was removed from the tripped condition. Technical Specification 3.3.2 Action Statement b required that the inoperable channel be placed in the tripped condition within 1 hour. With the high flow switch administratively inoperable and the trip function not in the tripped condition, Action Statement 3.3.2.b was no longer met and Technical Specification 3.0.3 was entered. Entry into Technical Specification 3.0.3 constitutes an operation prohibited by the Technical Specifications and is reportable per 10CFR50.73(a)(2)(i)(B). The required Operability testing was completed and Technical Specification 3.0.3 was exited at 1450 hours on 11/06/93. Replacing the high flow switch lower switch assembly did not create a significant degradation in the Station's ability to protect the health and safety of the public. The lower switch assembly was found with a worn micro switch actuating arm roller. Changes with respect to Technical Specification 3.0.3 will be pursued as part of the NRC generic Technical Specification Improvement Program.

LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 600 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)  Unit 2 Susquehanna Steam Electric Station	DOCKET NUMBER (2)  0   5   0   0   0   3   8   8	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		9   3	-   0   0   6	-   0   0	0   2	OF 0   3	

TEXT (If more space is required, use additional NRC Form 366A's) (17)

DESCRIPTION OF EVENT

On November 6, 1993, at 1323 hours with Unit 2 in Condition 1 at 100% power, the 'A' Main Steam Line high flow switch (FIS-B21-2N006B) (EIIS Code: JM) lower switch assembly was replaced due to erratic indication. At 0830 hours on 11/06/93 a Nuclear Plant Operator (Non-licensed, Utility) on rounds had found the flow switch indicating 45 psid when it should have indicated 57 psid. The Instrumentation and Controls (I&C) department was contacted to investigate the condition. A calibration of the instrument was performed and although the flow switch isolation setpoints were within acceptance criteria, I&C was not satisfied with the instrument's indication performance. The decision was made to replace the flow switch lower switch assembly. During replacement of the flow switch lower switch assembly, the trip function of the switch was placed in the tripped condition. At 1437 hours, in order to perform the subsequent functional and operability testing, the affected trip function was removed from the tripped condition. Technical Specification 3.3.2 Action Statement b requires that the inoperable channel be placed in the tripped condition within 1 hour. With the high flow switch administratively inoperable and the trip function not in the tripped condition, Action Statement 3.3.2.b was no longer met and Technical Specification 3.0.3 was entered. Entry into Technical Specification 3.0.3 constitutes an operation prohibited by the Technical Specifications and is reportable per 10CFR50.73(a)(2)(i)(B). The required Operability testing was completed and Technical Specification 3.0.3 was exited at 1450 hours on 11/06/93.

CAUSE OF EVENT

The cause of the event was due to mechanical binding of components within the flow switch lower switch assembly due to wear. Instrumentation and Control Technicians (Non-licensed, utility) found a micro switch actuating arm roller worn. This did not allow the arm roller to roll freely which in turn did not allow the flow switch dial to indicate the proper differential pressure. The flow switch isolation setpoints were unaffected by this condition.

REPORTABILITY/ANALYSIS

This event was determined to be reportable per 10CFR50.73(a)(2)(i)(B), as a condition prohibited by the plant's Technical Specifications in that Technical Specification 3.0.3 was entered when the 'A' Main Steam High Flow switch (FIS-B21-2N006B) lower switch assembly was taken out of the tripped condition required by Technical Specification 3.3.2 action b. Flow switch FIS-B21-2N006B was administratively inoperable between 1437 hours and 1450 hours on 11/06/93 while the required testing was being performed. The remaining flow switch within the same trip system and both flow switches in the remaining trip system were Operable throughout the time in which the 'B' switch was being tested. Therefore, taking flow switch FIS-B21-2N006B out of the tripped condition did not create a significant degradation in the Station's ability to protect the health and safety of the public and/or plant personnel. The worn roller arm found in the flow switch lower switch assembly did not adversely effect the trip function of the flow switch.

LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)  Unit 2 Susquehanna Steam Electric Station	DOCKET NUMBER (2)  0   5   0   0   0   3   8   8	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		9   3	-   0   0   6	-   0   0	0   3	OF 0   3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Concerning entry into Limiting Condition for Operation 3.0.3, the Technical Specifications contain no provision for allowing an isolation trip signal, imposed due to action requirements, to be reset to allow performance of Technical Specification required surveillances needed to restore the system to Operable status. Such provisions are being pursued as part of the NRC generic Technical Specification Improvement Program.

In accordance with guidance provided in NUREG 1022, Supplement 1 item 14.1 and 10CFR50.4(d), the required submission date for this report was determined to be 12/06/93.

CORRECTIVE ACTION

I&C investigated and subsequently replaced flow switch FIS-B21-2N006B lower switch assembly.

ADDITIONAL INFORMATION

Failed Component Identification:

Component - High Flow Switch FIS-B21-2N006B lower switch assembly

Model - 288A

Manufacturer - Barton

Past Similar Events:

A review of past Licensee Event Reports for the station identified three events involving entry into Technical Specification 3.0.3 in order to test pressure switches.

Docket No. 50-387 LER 90-009 Failed Reactor Pressure Switch

Docket No. 50-387 LER 91-001 Isolation Pressure Switch

Docket No. 50-387 LER 93-012 ECCS Low Reactor Pressure Permissive Switch