

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) <b>SUSQUEHANNA STEAM ELECTRIC STATION - UNIT 2</b>															DOCKET NUMBER (2) <b>0 5 0 0 0 3 8 8</b>					PAGE (3) <b>1 OF 0 3</b>	
TITLE (4) <b>OPERABILITY TESTING FOLLOWING INSTRUMENT LEAK REPAIR REQUIRED ENTRY INTO ICO 3.0.3</b>																					
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)											
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)								
0 5	2 4	9 3	9 3	0 0 4	0 0	0 6	2 3	9 3					0 5 0 0 0								
OPERATING MODE (9) <b>1</b>			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)																		
POWER LEVEL (10) <b>1 0 0</b>		20.402(b)				20.405(c)				50.73(a)(2)(iv)				73.71(b)							
		20.405(a)(1)(i)				50.38(c)(1)				50.73(a)(2)(v)				73.71(c)							
		20.405(a)(1)(ii)				50.38(c)(2)				50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)							
		20.405(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(viii)(A)											
		20.405(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(ix)(B)											
		20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(x)											
LICENSEE CONTACT FOR THIS LER (12)																					
NAME												TELEPHONE NUMBER									
Richard R. Wehry, Power Production Engineer - Compliance												AREA CODE <b>7 1 7</b>		<b>5 4 2 - 3 6 6 4</b>							
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																					
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		
SUPPLEMENTAL REPORT EXPECTED (14)														EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR			
YES (If yes, complete EXPECTED SUBMISSION DATE)														NO							

**ABSTRACT** (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On May 24, 1993, at 2305 hours, with Unit 2 in Condition 1 at 100% power, repairs were completed on a leaking fitting on the reference leg of reactor vessel level switch LIS-24221C. This level switch provides signals to the containment isolation logic for Drywell Cooling, Reactor Building Closed Cooling Water and Containment Instrument Gas systems. In order to perform the required functional and operability testing, the affected trip system was taken out of the tripped condition required by Technical Specification 3.3.2 ACTION b. With the trip system out of the tripped condition, ACTION b. was no longer met and Technical Specification 3.0.3 was entered. Entry into Technical Specification 3.0.3 constitutes an operation prohibited by the Technical Specifications and is reportable per 10CFR50.73(a)(2)(i)(B). Since redundant instrumentation for Drywell Cooling Isolation Logic remained operable, there were no safety consequences or compromise to public health or safety as a result of the entry into LCO 3.0.3. This condition was cleared following return of the switch to operable status. Changes with respect to Technical Specification 3.0.3 are being pursued as part of the NRC generic Technical Specification Improvement Program.

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PDR ADOCK 05000388  
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LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION

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FACILITY NAME (1)  SUSQUEHANNA STEAM ELECTRIC STATION - UNIT 2	DOCKET NUMBER (2)  0 5 0 0 0 3 8 8	LER NUMBER (6)			PAGE (3)		
		YEAR 9 3	SEQUENTIAL NUMBER 0 0 4	REVISION NUMBER 0 0	0 2 OF 0 3		

TEXT (If more space is required, use additional NRC Form 366A's) (17)

DESCRIPTION OF EVENT

On May 24, 1993 at 2305 hours, with Unit 1 in Condition 2 at 100% power, repairs were completed on a leaking fitting on the reference leg of reactor vessel water level switch LIS-24221C (EIIS Code: JM). This level switch provides signals to the containment isolation logic for Drywell Cooling (EIIS Code: VB), Reactor Building Closed Cooling Water (EIIS Code: CC), and Containment Instrument Gas (EIIS Code: LD) systems. In order to perform the required functional and operability testing, the affected trip system was taken out of the tripped condition required by Technical Specification 3.3.2 ACTION b. for performing the repairs. With the trip system out of the tripped condition, ACTION b. was no longer met and Technical Specification Limiting Condition for Operation (LCO) 3.0.3 was entered. Entry into LCO 3.0.3 constitutes an operation prohibited by the plant's Technical Specifications and is reportable per 10CFR50.73(a)(2)(i)(B).

CAUSE OF EVENT

The cause for entry into LCO 3.0.3 ACTION was the inability to meet Technical Specification 3.3.2 ACTION b. when the affected trip system had to be taken out of the tripped condition to perform the required functional and operability testing following leak repairs. The leak was at an elbow fitting on the reference leg of reactor vessel water level switch LIS-24221C. Because of heightened awareness of the potential effects of reference leg leaks on reactor water level instrumentation signals (Generic Letter 92-04 and NRC Bulletin 93-03), the need to repair the leak and enter the LCO 3.0.3 ACTION for operability testing was reinforced.

REPORTABILITY/ANALYSIS

This event was determined reportable per 10CFR50.73(a)(2)(i)(B), in that Susquehanna Unit 2 was in an operating condition prohibited by the plant's Technical Specifications (LCO 3.0.3) while functional and operability testing was performed on reactor vessel water level switch LIS-24221C following the repair of a leaking fitting on the switch's reference leg side. LIS-24221C provides signals to the containment isolation logic for Drywell Cooling (EIIS Code: VB), Reactor Building Closed Cooling Water (EIIS Code: CC), and Containment Instrument Gas (EIIS Code: LD) systems.

LCO ACTION 3.0.3 was entered at 2305 hours on May 24, 1993 and was cleared at 0002 hours on May 25, 1993, following successful functional and operability testing.

LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION

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		YEAR 93	SEQUENTIAL NUMBER 004	REVISION NUMBER 00	03 OF 03		

TEXT (If more space is required, use additional NRC Form 366A's) (17)

There were no safety consequences or compromise to public health or safety as a result of the entry into LCO 3.0.3. Redundant instrumentation for Drywell Cooling Isolation remained operable during the time that LIS-24221C was being tested for restoring it to operable status. The time that the unit was in LCO ACTION 3.0.3 was minimized to the extent practicable.

In accordance with the guidance provided in NUREG 1022, Supplement 1, item 14.1, the required submission date for this report was determined to be June 23, 1993.

CORRECTIVE ACTION

Following functional and operability testing of level switch LIS-24221C, the switch was declared operable at 0002 hours on May 25, 1993, and LCO ACTION 3.0.3 was cleared. The leak at the elbow fitting was considered to be an isolated incident.

PP&L is pursuing changes with respect to Technical Specification 3.0.3 as part of the NRC generic Technical Specification Improvement Program.

ADDITIONAL INFORMATION

Failed Component Identification: Not applicable.

Previous Similar Events: LER 50-387/93-002-00.

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