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ACCESSION NBR: 9302250306      DOC. DATE: 93/02/22      NOTARIZED: NO      DOCKET #  
 FACIL: 50-387 Susquehanna Steam Electric Station, Unit 1, Pennsylvania      05000387  
 AUTH. NAME      AUTHOR AFFILIATION  
 WEHRY, R.R.      Pennsylvania Power & Light Co.  
 STANLEY, H.G.      Pennsylvania Power & Light Co.  
 RECIP. NAME      RECIPIENT AFFILIATION

SUBJECT: LER 93-002-00: on 930122, instrument fitting leak repair  
 required entry into LCO 3.0.3 due to inability to meet ECCS  
 TS LCO 3.5.1. Water level switch LIS-B21-1N031D reworked &  
 replaced. W/930222 ltr.

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 TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc.

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**Pennsylvania Power & Light Company**

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February 22, 1993

U.S. Nuclear Regulatory Commission  
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Washington, DC 20555

SUSQUEHANNA STEAM ELECTRIC STATION  
LICENSEE EVENT REPORT 93-002-00  
FILE R41-2  
PLAS - 561

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Docket No. 50-387  
License No. NPF-14

Attached is Licensee Event Report 93-002-00. This report is being made pursuant to 10CFR50.73(a)(2)(i)(B) in that Susquehanna Unit 1 was in an operating condition prohibited by the plant's Technical Specifications while a leaking tubing fitting on a reactor water level switch was repaired. Repair of this leak required entry into Limiting Condition for Operation 3.0.3 because the Technical Specification actions could not be met. The condition was cleared following repairs.

H.G. Stanley  
Superintendent of Plant - Susquehanna

RRW/mjm

cc: Mr. T. T. Martin  
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9302250306 930222  
PDR ADCK 05000387  
S PDR

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LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 500 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) <b>Susquehanna Steam Electric Station - Unit 1</b>	DOCKET NUMBER (2) <b>0 5 0 0 0 3 8 7</b>	PAGE (3) <b>1 OF 0 3</b>
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TITLE (4)  
**Instrument Fitting Leak Repair Required Entry into LCO 3.0.3**

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0	1	2 2 9 3	9 3	0 0 2	0 0	0 2	2 2	9 3			0 5 0 0 0

OPERATING MODE (9) <b>1</b>	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)									
POWER LEVEL (10) <b>1 1 0 1 0</b>	20.402(b)	20.405(e)	50.73(a)(2)(iv)	73.71(b)						
	20.405(a)(1)(i)	50.38(e)(1)	50.73(a)(2)(v)	73.71(c)						
	20.405(a)(1)(ii)	50.38(e)(2)	50.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)						
	20.405(a)(1)(iii)	<input checked="" type="checkbox"/> 50.73(a)(2)(i)	50.73(a)(2)(viii)(A)							
	20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)							
20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(a)								

LICENSEE CONTACT FOR THIS LER (12)							TELEPHONE NUMBER			
NAME <b>Richard R. Wehry - Power Production Engineer - Compl.</b>							AREA CODE <b>7 1 7</b>			
							<b>5 4 2 7 3 6 1 4</b>			

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)										
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)				EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)				<input checked="" type="checkbox"/> NO			

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

At 1255 hours on January 22, 1993, with Unit 1 in Condition 1 at 100% power, repair of a leak on the valve manifold for reactor water level switch LIS-B21-1N031D necessitated entering Technical Specification LCO 3.0.3. The leaking fitting was on the reference leg side of the reactor water level instrument. Because of recent heightened awareness of the effects of reference leg leaks on reactor water level indication (Generic Letter 92-04) and radiological contamination concerns, the need to repair the leak and enter the LCO 3.0.3 ACTION was reinforced. LIS-B21-1N031D provides initiation signals to Core Spray Division II, Residual Heat Removal Low Pressure Coolant Injection (RHR LPCI) Division II, High Pressure Coolant Injection (HPCI), Automatic Depressurization System (ADS) and Reactor Core Isolation Cooling (RCIC; non-ECCS system). The ACTION of Technical Specification LCO 3.3.3.b requires that the associated inoperable trip system be placed in the tripped condition within 1 hour or declare the associated ECCS inoperable. Since placing the trip system in the tripped condition would result in the actuation of the ECCS systems, the associated ECCS systems were declared inoperable, which required entry into LCO 3.0.3 when the ECCS Technical Specification 3.5.1 actions could not be met. Repairs were made and LCO 3.0.3 was cleared at 1523 hours on January 22, 1993.

LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)  Unit 1 Susquehanna Steam Electric Station	DOCKET NUMBER (2)  0 5   0   0   0   3   8   7 9   3	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		9   3	-   0   0   2	-   0   0	0   2	OF	0   3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

DESCRIPTION OF EVENT

At 1255 hours on January 22, 1993, with Unit 1 in Condition 1 at 100% power, repair of a leaking tubing fitting on the valve manifold for reactor water level switch LIS-B21-1N031D (EIS Code: I) necessitated entering Technical Specification Limiting Condition for Operation (LCO) 3.0.3. LIS-B21-1N031D provides initiation signals to emergency core cooling systems (ECCS) Core Spray Division II (EIS Code: BM), Residual Heat Removal Low Pressure Coolant Injection (RHR LPCI; EIS Code: BO) Division II, High Pressure Coolant Injection (HPCI; EIS Code: BJ) and Automatic Depressurization System (ADS; EIS Code: SB) and to the non-ECCS Reactor Core Isolation Cooling System (RCIC; EIS Code: BN).

Taking LIS-B21-1N031D out of service to repair the leak, required taking the action of Technical Specification LCO 3.3.3.b. LCO ACTION 3.3.3.b requires that the associated inoperable trip system be placed in the tripped condition within 1 hour or declare the associated ECCS inoperable. Since placing the trip system in the tripped condition would result in the actuation of the aforementioned ECCS systems, the associated ECCS systems were declared inoperable, which required entry into LCO 3.0.3 ACTION when the ECCS Technical Specification LCO 3.5.1 ACTIONS could not be met. The leaking fitting was on the reference leg side of the reactor water level instrument. Because of recent heightened awareness of the effects of reference leg leaks on reactor water level indication (Generic Letter 92-04) and radiological contamination concerns, the need to repair the leak and enter the LCO 3.0.3 ACTION was reinforced. The time in which the unit was in the LCO 3.0.3 ACTION was minimized to the extent practicable and the condition was cleared at 1523 hours on January 22, 1993.

CAUSE OF EVENT

The cause for entry into LCO 3.0.3 ACTION was the inability to meet the ECCS Technical Specification LCO 3.5.1 required ACTIONS when a leaking tubing fitting on the valve manifold for reactor water level switch LIS-B21-1N031D required repair. The leaking fitting was on the reference leg side of the reactor water level instrument. Because of recent heightened awareness of the effects of reference leg leaks on reactor water level indication (Generic Letter 92-04) and radiological contamination concerns, the need to repair the leak and enter the LCO 3.0.3 ACTION was reinforced.

REPORTABILITY/ANALYSIS

This event was determined reportable per 10CFR50.73(a)(2)(i)(B) in that Susquehanna Unit 1 was in an operating condition prohibited by the plant's Technical Specifications (namely LCO 3.0.3) while a leaking tubing fitting on reactor water level switch LIS-B21-1N031D valve manifold was repaired.

LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 60.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)  Unit 1 Susquehanna Steam Electric Station	DOCKET NUMBER (2)  0   5   0   0   0   3   8   7	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		9   3	-   0   0   2	-   0   0	0   3	OF	0   3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

LIS-B21-1N031D provides initiation signals to ECCS systems Core Spray Division II, RHR LPCI Division II, HPCI and ADS and non-ECCS RCIC. Taking LIS-B21-1N031D out of service required entry into LCO 3.3.3.b which requires that the associated inoperable trip system be placed in the tripped condition within 1 hour or declare the associated ECCS inoperable. Since placing the trip system in the tripped condition would result in initiation of the aforementioned ECCS systems, the associated ECCS systems were declared inoperable, which required entry into LCO 3.0.3 ACTION when the actions of ECCS Technical Specification LCO 3.5.1 could not be met.

LCO ACTION 3.0.3 was entered at 1255 hours on January 22, 1993 and was cleared at 1523 hours on January 22, 1993, following repair of the valve manifold leaking tubing fitting.

There were no safety consequences or compromise to public health or safety as a result of the entry into LCO 3.0.3. Redundant instrumentation for initiation of Core Spray, RHR LPCI, HPCI, ADS and RCIC remained operable during the time that LIS-B21-1N031D was out of service and all ECCS remained capable of performing their design safety function during this time. The time that the unit was in LCO ACTION 3.0.3 was minimized to the extent practicable.

In accordance with the guidance provided in NUREG 1022, Supplement 1, item 14.1 and 10CFR50.4(d), the required submission date for this report was determined to be February 22, 1993.

CORRECTIVE ACTIONS

All tubing fittings on the valve manifold for reactor water level switch LIS-B21-1N031D were reworked and replaced. The manifold was then pressurized to 1000 psig, no leaks were observed and the instrument was returned to service.

ADDITIONAL INFORMATION

Failed Component Identification: N/A

Previous Similar Events: No previous similar reports have been issued relative to entry into LCO 3.0.3 for a leak repair.