

# ACCELERATED DISTRIBUTION DEMONSTRATION SYSTEM

## REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 9203170278      DOC. DATE: 92/01/10      NOTARIZED: NO      DOCKET #  
 FACIL: 50-388 Susquehanna Steam Electric Station, Unit 2, Pennsylvania      05000388  
 AUTH. NAME      AUTHOR AFFILIATION  
 METER, J. J.      Pennsylvania Power & Light Co.  
 STANELY, H. G.      Pennsylvania Power & Light Co.  
 RECIP. NAME      RECIPIENT AFFILIATION

SUBJECT: LER 91-014-00: on 921212, MSIV closure time exceeded TS limit.  
 Caused by Response Time Testing procedures not accounting  
 for Trip Channel Sensor. Changes were initiated for I&C  
 Response Time Testing procedures. W/920227 ltr.

DISTRIBUTION CODE: IE22T      COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 2  
 TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc.

NOTES: LPDR 1 cy Transcripts.

05000388

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	AEOD/ROAB/DSP	2 2	NRR/DET/EMEB 7E	1 1
	NRR/DLPQ/LHFB10	1 1	NRR/DLPQ/LPEB10	1 1
	NRR/DOEA/OEAB	1 1	NRR/DREP/PRPB11	2 2
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	REG <del>FILE</del> 02	1 1	RES/DSIR/EIB	1 1
	RGNI <del>FILE</del> 01	1 1		
EXTERNAL:	EG&G BRYCE, J. H.	3 3	L ST LOBBY WARD	1 1
	NRC PDR	1 1	NSIC MURPHY, G. A.	1 1
	NSIC POORE, W.	1 1	NUDOCS FULL TXT	1 1
NOTES:		2 2		

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*A04*

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**Pennsylvania Power & Light Company**

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February 27, 1992

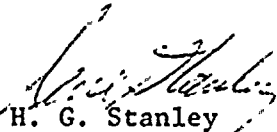
U.S. Nuclear Regulatory Commission  
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SUSQUEHANNA STEAM ELECTRIC STATION  
CORRECTION TO LICENSEE EVENT REPORT 91-014-00  
PLAS - 516 FILE R41-2

Docket No. 50-388  
License No. NPF-22

Dear Sir:

Please replace page one of four in Licensee Event Report (LER) 91-014-00 (Docket No. 50-388, License No. NPF-22) with the attached. The title was omitted in error.



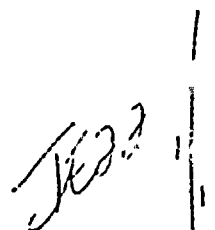
H. G. Stanley  
Superintendent of Plant - Susquehanna

JJM/mjm

cc: Mr. T. T. Martin  
Regional Administrator, Region I  
U.S. Nuclear Regulatory Commission  
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P.O. Box 35  
Berwick, PA 18603-0035

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PDR ADDCK 05000388  
S PDR





LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Susquehanna Steam Electric Station - Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 3 8 8	PAGE (3) 1 OF 0 4
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TITLE (4)  
MSIV Closure Response Time Exceeded Technical Specification Limit

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
1	2	9	1	0	0	0	1	1			0 5 0 0 0
1	2	9	1	0	0	0	1	1			0 5 0 0 0

OPERATING MODE (9) 1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)										
POWER LEVEL (10) 1 0 0	20.402(b)			20.405(c)			50.73(a)(2)(iv)			73.71(b)	
	20.405(a)(1)(i)			50.38(c)(1)			50.73(a)(2)(v)			73.71(c)	
	20.405(a)(1)(ii)			50.38(c)(2)			50.73(a)(2)(vii)			OTHER (Specify in Abstract below and in Text, NRC Form 365A)	
	20.405(a)(1)(iii)			50.73(a)(2)(i)			50.73(a)(2)(viii)(A)				
	20.405(a)(1)(iv)			50.73(a)(2)(ii)			50.73(a)(2)(viii)(B)				
	20.405(a)(1)(v)			50.73(a)(2)(iii)			50.73(a)(2)(ix)				

LICENSEE CONTACT FOR THIS LER (12)									
NAME J. J. Meter - Compliance Engineer							TELEPHONE NUMBER 7 1 7 5 4 2 - 1 8 7 3		

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)										
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)						EXPECTED SUBMISSION DATE (15)		
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)				<input checked="" type="checkbox"/> NO		MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

At 1400 hours on December 12, 1991, after reviewing an industry event report, an Instrumentation and Control Engineer discovered that a Main Steam Isolation Valve (MSIV) Closure Channel Response Time had exceeded the Technical Specification required value of 60 milli-seconds (msec) by 2 msec. The Unit 2 "B1" channel was 2 msec over Technical Specification limits from the Fall of 1989 until the Spring of 1991. The cause of the event was that Response Time Testing procedures did not account for the Trip Channel Sensor (limit switch contact) Response time of 10 msec as identified in General Electric Design Data sheets. Therefore the measured Response Time of 52 msec was actually 62 msec. The remaining Closure Response Channels were within Technical Specification limits and the system would have performed its intended function. Changes were initiated for I&C Response Time Testing procedures to ensure that 10 msec for Channel Sensor Response is included in the overall RPS Response Time for MSIV Closure and Turbine Stop Valve Closure. The reason why the Sensor Response Time allotment was not accounted for could not be determined. A review of Response Time procedures of both Units was performed to ensure no other Response Time tests were affected by the General Electric Design Data Sheets. No safety consequences or compromises to the health or safety of the public were attributed to the event.