

ACCELERATED DISTRIBUTION DEMONSTRATION SYSTEM

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 9109040328 DOC. DATE: 91/08/28 NOTARIZED: NO DOCKET #
 FACIL: 50-387 Susquehanna Steam Electric Station, Unit 1, Pennsylv 05000387
 AUTH. NAME AUTHOR AFFILIATION
 LLOYD, H. Pennsylvania Power & Light Co.
 STANLEY, H.G. Pennsylvania Power & Light Co.
 RECIP. NAME RECIPIENT AFFILIATION

SUBJECT: LER 91-009-00: on 910731, main steam line radiation monitor B failed, resulting in actuation of Div B reactor protection & main steam isolation sys. Caused by failure of capacitor in monitor power supply. Monitor repaired. W/910828 ltr.

DISTRIBUTION CODE: IE22T COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 4
 TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc.

NOTES: LPDR 1: cy Transcripts. 05000387

RECIPIENT ID CODE/NAME	COPIES LTR	ENCL	RECIPIENT ID CODE/NAME	COPIES LTR	ENCL
PD1-2 IA	1	1	PD1-2 PD	1	1
THADANI, M.	1	1			

INTERNAL:	RECIPIENT ID CODE/NAME	COPIES LTR	ENCL	RECIPIENT ID CODE/NAME	COPIES LTR	ENCL
	ACNW	2	2	ACRS	2	2
	AEOD/DOA	1	1	AEOD/DSP/TPAB	1	1
	AEOD/ROAB/DSP	2	2	NRR/DET/ECMB 9H	1	1
	NRR/DET/EMEB 7E	1	1	NRR/DLPQ/LHFB10	1	1
	NRR/DLPQ/LPEB10	1	1	NRR/DOEA/OEAB	1	1
	NRR/DREP/PRPB11	2	2	NRR/DST/SELB 8D	1	1
	NRR/DST/SICB8H3	1	1	NRR/DST/SPLB8D1	1	1
	NRR/DST/SRXB 8E	1	1	REG FILE 02	1	1
	RES/DSIR/EIB	1	1	RGN1 FILE 01	1	1
EXTERNAL:	EG&G BRYCE, J.H	3	3	L ST LOBBY WARD	1	1
	NRC PDR	1	1	NSIC MURPHY, G.A	1	1
	NSIC POORE, W.	1	1	NUDOCS FULL TXT	1	1

NOTES: 2 2

NOTE TO ALL "RIDS" RECIPIENTS:

PLEASE HELP US TO REDUCE WASTE! CONTACT THE DOCUMENT CONTROL DESK, ROOM P1-37 (EXT. 20079) TO ELIMINATE YOUR NAME FROM DISTRIBUTION LISTS FOR DOCUMENTS YOU DON'T NEED!

FULL TEXT CONVERSION REQUIRED
 TOTAL NUMBER OF COPIES REQUIRED: LTR 35 ENCL 35



Pennsylvania Power & Light Company

Two North Ninth Street • Allentown, PA 18101 • 215 / 770-5151

August 28, 1991

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

SUSQUEHANNA STEAM ELECTRIC STATION
LICENSEE EVENT REPORT 91-009-00
FILE R41-2
PLAS - 496

Docket No. 50-387
License No. NPF-14

Attached is Licensee Event Report 91-009-00. This report is being made pursuant to 10CFR50.73(a)(2)(i)(B), in that Susquehanna Unit 1 was in an operating condition prohibited by the Technical Specifications while testing was performed on a Main Steam Line Radiation Monitor following maintenance. This condition was cleared once the monitor was restored to operable status.

H.G. Stanley
Superintendent of Plant - Susquehanna

HL/mjm

cc: Mr. T. T. Martin
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Mr. G. S. Barber
Sr. Resident Inspector
U.S. Nuclear Regulatory Commission
P.O. Box 35
Berwick, PA 18603-0035

C
9109040328 910828
PDR ADOCK 05000387
S PDR

JE27

LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Susquehanna Steam Electric Station - Unit 1							DOCKET NUMBER (2) 0 5 0 0 0 3 8 7			PAGE (3) 1 OF 0 3	
--	--	--	--	--	--	--	--------------------------------------	--	--	----------------------	--

TITLE (4)
Operation Prohibited by Technical Specification During Radiation Monitor Retest

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)			
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)	
0	7	3	1	9	1	9	1	9	1	0	0	0
				0	0	9		0	0	0	8	2
						0	8	2			8	9
											0	5
											0	0

OPERATING MODE (8) 1		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)									
POWER LEVEL (10) 1 0 0		20.402(b)		20.405(c)		50.73(a)(2)(iv)		73.71(b)			
		20.405(a)(1)(i)		50.38(c)(1)		50.73(a)(2)(v)		73.71(c)			
		20.405(a)(1)(ii)		50.38(c)(2)		50.73(a)(2)(vii)		OTHER (Specify in Abstract below and in Text, NRC Form 366A)			
		20.405(a)(1)(iii)	X	50.73(a)(2)(i)		50.73(a)(2)(viii)(A)					
		20.405(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)					
		20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(x)					

LICENSEE CONTACT FOR THIS LER (12)									
NAME Harrison Lloyd, Jr. - Power Production Engineer							TELEPHONE NUMBER 7 1 7 5 4 2 1 - 3 9 1 7		

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)											
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		

SUPPLEMENTAL REPORT EXPECTED (14)							EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE) <input checked="" type="checkbox"/> NO											

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On July 31, 1991 with Unit 1 operating at 100% power, the 'B' Main Steam Line Radiation Monitor failed resulting in actuation of the 'B' Division of the Reactor Protection and Main Steam Isolation Systems. Investigation revealed that a capacitor in the monitor power supply had failed causing the loss of the monitor and the associated trips. The defective capacitor was replaced and the power supply for the monitor was restored. In order to perform the post-maintenance required surveillance, the affected channel had to be reset (i.e. - taken out of the tripped condition). With the channel taken out of the tripped condition, the ACTION Statement was no longer met and as a result, Technical Specification 3.0.3 was entered. The surveillance test was then successfully completed. The entry into Technical Specification 3.0.3 constituted operation prohibited by the Technical Specification since they do not recognize or allow this action of removing Technical Specification imposed trips from the tripped condition for performance of post-maintenance surveillance testing. This event was determined to be reportable per 10CFR50.73(a)(2)(i)(B) as an operation prohibited by Technical Specifications. There were no safety consequences or compromises since only one channel of one division of the trip system was affected and since it failed to the safe (tripped) condition. The defective radiation monitor was repaired and the logic channel was tested satisfactorily. The power supply failure was attributed to a capacitor failed in a power supply card. We are pursuing a change to our Technical Specification to allow resetting trip signals in order to perform required surveillance testing following maintenance.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 60.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Unit 1 Susquehanna Steam Electric Station	DOCKET NUMBER (2) 0 5 0 0 0 3 8 7 9 1	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		9 1	0 0 9	0 0	0 2	OF	0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

DESCRIPTION OF EVENT

On July 31, 1991 at 0755 hours with Unit 1 in Condition 1 at 100% power, the 'B' Main Steam Line Radiation Monitor failed resulting in actuation of the 'B' Division of the Reactor Protection System (EIIIS Code: JC) and the 'B' Division of the Main Steam Isolation Valve Isolation Logic (EIIIS Code: JM). The Main Steam Line Radiation Monitor recorder displayed erratic indication of the 'B' channel but steady and normal indication for the A,C & D. The power indicating light for the 'B' monitor was extinguished, indicating a possible power supply problem. The affected channel was declared inoperable and was placed in the tripped condition as required by Technical Specification 3.3.2, Action Statement B. Investigation revealed that a capacitor in the monitor power supply had failed causing the loss of the monitor and the associated trips. The defective capacitor was replaced and the power supply for the monitor was restored. In order to perform the post-maintenance required surveillance, the affected channel had to be reset (i.e. - taken out of the tripped condition). With the channel taken out of the tripped condition, the ACTION Statement was no longer met and as a result, Technical Specification 3.0.3 was entered. The surveillance test was then successfully completed and the 'B' Main Steam Line Radiation monitor was declared operable. The entry into Technical Specification 3.0.3. constituted operation prohibited by the Technical Specification since they do not recognize or allow this action of removing Technical Specification imposed trips from the tripped condition for performance of post-maintenance surveillance testing. This condition existed from 2041 hours until 2200 hours.

CAUSE OF EVENT

The cause of the radiation monitor failure and associated trips was the failure of a capacitor in the monitor power supply. The capacitor failure was determined to be an isolated failure of an electronic component.

REPORTABILITY/ANALYSIS

This event was determined to be reportable per 10CFR50.73(a)(2)(i)(B), in that Technical Specification 3.3.2 action b., was not met while the 'B' Main Steam Line Rad Monitor was being tested. The affected channel had to be reset to allow surveillance testing to restore OPERABILITY of the channel. As Technical Specifications does not recognize or allow this action, LCO 3.0.3 was entered. This 3.0.3 entry constituted operation prohibited by the Technical Specification (NUREG 1022, Supp. 1, item 2.4).

Taking the channel out of the tripped condition did not create a significant degradation in our ability to protect the health and safety of the public and/or plant personnel since only one of the two channels in the Division 2 trip system was inoperable. The other channel in the Division 2 trip system and both channels of the Division 1 trip system were operable while this condition existed. This event would not have been more significant at any other operating condition.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Unit 1 Susquehanna Steam Electric Station	DOCKET NUMBER (2) 0 5 0 0 0 3 8 7	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		9 1	- 0 0 9	- 0 0	0 3	OF	0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

When the rad monitor was determined to be inoperable, its associated channel was placed in the tripped condition. Once the rad monitor was repaired, the trip had to be reset to facilitate testing necessary to ensure it would provide the required trip function. The action statement was not met only during the time period that the defective instrument was being tested. This time period was approximately one and a half hours.

In accordance with guidance provided in NUREG 1022, Supplement 1 item 14.1 and 10CFR50.4(d); the required submission date for this report was determined to be 08/30/91.

CORRECTIVE ACTION

The defective radiation monitor was repaired and the logic channel was tested satisfactorily. The power supply failure was attributed to a capacitor failure in a power supply card.

Concerning entry into Limiting Condition for Operation 3.0.3., the Technical Specifications contain no provision for allowing an isolation trip signal, imposed due to action requirements, to be reset to allow performance of Technical Specification required surveillances needed to restore the instrument to OPERABLE status. Such provisions are considered necessary and are being pursued as a change to our Technical Specification.

ADDITIONAL INFORMATION

Failed Component Identification: N/A

Previously Reported Events with similar results but with dissimilar causes:

Docket No. 50-387 LER 90-009 Failed Reactor Pressure Switch - Operation Prohibited by Technical Specification during Retest.

Docket No. 50-387 LER 91-001 Isolation Pressure Switch - Operation Prohibited by Technical Specification during Retest.

Docket No. 50-387 LER 91-003 Failed Relay in the MSIV Isolation Logic - Operation Prohibited by Technical Specification during Retest.