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REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 9010260063 DOC. DATE: 90/10/19 NOTARIZED: NO DOCKET #
 FACIL: 50-387 Susquehanna Steam Electric Station, Unit 1, Pennsylv 05000387
 AUTH. NAME AUTHOR AFFILIATION
 CRIST, M.L. Pennsylvania Power & Light Co.
 STANLEY, H.G. Pennsylvania Power & Light Co.
 RECIP. NAME RECIPIENT AFFILIATION

SUBJECT: LER 90-020-00: on 900920, "As Found" main steam line penetration leakage rate exceeds Tech Spec limits.

DISTRIBUTION CODE: IE22T COPIES RECEIVED: LTR / ENCL / SIZE: 4
 TITLE: 50.73/50.9 Licensee Event Report (LER), Incident/Rpt, etc.

NOTES: LPDR 1 cy Transcripts. 05000387 /

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INTERNAL:	ACNW		2	2		ACRS		2	2
	AEOD/DOA		1	1		AEOD/DSP/TPAB		1	1
	AEOD/ROAB/DSP		2	2		NRR/DET/ECMB 9H		1	1
	NRR/DET/EMEB 7E		1	1		NRR/DLPQ/LHFB11		1	1
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	REG FILE 02		1	1		RES/DSIR/EIB		1	1
	RGNI FILE 01		1	1					
EXTERNAL:	EG&G BRYCE, J.H		3	3		L ST LOBBY WARD		1	1
	NRC PDR		1	1		NSIC MAYS, G		1	1
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NOTES: 2 2

NOTE TO ALL "RIDS" RECIPIENTS:

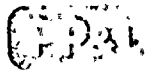
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Pennsylvania Power & Light Company

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October 19, 1990

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

SUSQUEHANNA STEAM ELECTRIC STATION
LICENSEE EVENT REPORT 90-020-00
FILE R41-2
PLAS -447

Docket No. 50-387
License No. NPF-14

Attached is Licensee Event Report 90-020-00. This event was determined to be reportable per 10CFR50.73(a)(2)(ii) in that the Main Steam Line penetration leakage exceeded the Technical Specification limit during regularly scheduled Local Leak Rate Testing.

H.G. Stanley
Superintendent of Plant - Susquehanna

MLC/mjm

cc: Mr. T.T. Martin
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Mr. G.S. Barber
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LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Susquehanna Steam Electric Station - Unit 1		DOCKET NUMBER (2) 0 5 0 0 0 3 8 7 1	PAGE (3) 1 OF 0 3
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"As Found" Main Steam Line Penetration Leakage Rate Exceeds Technical Specification Limits

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0 9	2 0	9 0	9 0	0 2 0	0 0	1 0	1 9	9 0			0 5 0 0 0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

OPERATING MODE (9) 5	20.402(b)	20.405(c)	60.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10) 0 1 0 0	20.406(a)(1)(i)	60.38(c)(1)	60.73(a)(2)(v)	73.71(c)
	20.406(a)(1)(ii)	60.38(c)(2)	60.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
	20.406(a)(1)(iii)	60.73(a)(2)(ii)	60.73(a)(2)(viii)(A)	
	20.405(a)(1)(iv)	60.73(a)(2)(iii)	60.73(a)(2)(viii)(B)	
	20.406(a)(1)(v)	60.73(a)(2)(iii)	60.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME Michael L. Crist - Compliance Evaluator	TELEPHONE NUMBER 7 1 1 7 5 1 4 2 1 - 3 2 1 8 9
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
X	SIB	IISIV	A151815	Y					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single space typewritten lines) (16)

At 1230 hours on September 20, 1990 with Unit 1 in its fifth refueling and inspection outage, an evaluation of data from the scheduled Main Steam Line (MSL) penetration Local Leak Rate Tests (LLRTs) determined that the "as found" leakage was in excess of the limit of Technical Specification 3.6.1.2(c) for the total MSL containment penetration leakage of 46.0 standard cubic feet per hour (scfh). The "as found" minimum path leakage rate was 110.9 scfh. The C MSL containment penetration accounted for 99.6 scfh of the total penetration leakage. In accordance with 10CFR50.72(b)(2)(i) an ENS 4 hour non-emergency call was made at 1355 hours.

The C MSL inboard and outboard Main Steam Isolation Valves were reworked and a post maintenance LLRT was performed with satisfactory results. The inspection only revealed slight areas of light oxidation and minor surface scratching, neither of which are considered contributors to the cause of the high "as found" leakage rate. Total MSL containment penetration leakage was reduced to 20.9 scfh. There was no safety significance or risk to the health and safety of the public due to this event.



LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 60.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-630), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3160-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Unit 1 Susquehanna Steam Electric Station	DOCKET NUMBER (2) 0 5 0 0 0 3 8 7 9 0	LER NUMBER (8)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		0	2	0	0	2	0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

EVENT DESCRIPTION

At 1230 hours on September 20, 1990 with Unit 1 in its fifth refueling and inspection outage, evaluation of data from the scheduled Main Steam Line (MSL, EIIIS Code: SB) penetration Local Leak Rate Tests (LLRTs) determined that the "as found" leakage was in excess of the limit of Technical Specification 3.6.1.2(c) for the total MSL containment penetration leakage of 46.0 standard cubic feet per hour (scfh). The total "as found" MSL containment penetration minimum path leakage rate was 110.9 scfh. The C MSL containment penetration accounted for 99.6 scfh of the total penetration leakage. The leakage for the A, B and D MSL penetrations was 0.6, 3.2 and 7.5 scfh respectively. In accordance with 10CFR50.72(b)(2)(i) an ENS 4 hour non-emergency call was made at 1355 hours.

CAUSE OF EVENT

The high leakage rate on the C MSL containment penetration was attributed to the performance of the inboard and outboard Main Steam Line Isolation Valves (MSIVs), HV-141F022C and HV-141F028C. The valves were disassembled and inspected in conformance with the detailed visual and dimensional criteria developed by the BWR Owners Group's MSIV Subgroup. The inspection only revealed slight areas of light oxidation and minor surface scratching, neither of which are considered contributors to the cause of the high "as found" leakage rate. This is not an uncommon anomaly when reworking MSIVs.

REPORTABILITY/ANALYSIS

This event was determined to be reportable under 10CFR50.73(a)(2)(ii) in that the excessive leakage through both inboard and outboard valves in the C MSL resulted in MSL containment penetration leakage which was in excess of Technical Specification limits.

An assessment of the safety consequences associated with this event has been performed by Nuclear Plant Engineering. The assessment took into account the availability of the MSIV Leakage Control System (MSIV-LCS). The MSIV-LCS is designed to control and minimize the possible release of radioactive gases which could leak through the closed MSIVs following a LOCA. The "as found" MSL penetration leakage was within the capacity of the MSIV-LCS. As such, there was no safety significance or risk to the health and safety of the public due to this event.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 60.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Unit 1 Susquehanna Steam Electric Station	DOCKET NUMBER (2) 0 5 0 0 0 3 8 7	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		9 0	- 0 2 0	- 0 0	0 3	OF 0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

In accordance with the guidance provided in NUREG 1022 Supplement 1 Items 14.1 and 14.10, the required submission date for this report was determined to be October 22, 1990.

CORRECTIVE ACTIONS

The C MSL inboard and outboard MSIVs were disassembled and inspected. Following routine maintenance the valves were re-assembled and a successful post maintenance LLRT was performed. Leakage for the C MSL penetration was reduced to 0.38 scfh. Total MSL penetration leakage was reduced to 20.9 scfh.

ADDITIONAL INFORMATION

Past Similar Events: LER 83-062-00, Docket No. 387/License No. NPF-14
LER 83-064-00, Docket No. 387/License No. NPF-14
LER 89-010-01, Docket No. 388/License No. NPF-22

Failed Component: MSIVs, HV-141F022C and HV-141F028C

Manufacturer: Atwood and Morrill Co., Inc.

Model: A&M No. 21190-H