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ACCESSION NBR: 9003220265 DOC. DATE: 90/03/14 NOTARIZED: NO DOCKET #
 FACIL: 50-387 Susquehanna Steam Electric Station, Unit 1, Pennsylvania 05000387
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 RECIP. NAME RECIPIENT AFFILIATION

SUBJECT: LER 90-009-00: on 900214, failed reactor pressure switch operation prohibited by Tech Spec during retest.

DISTRIBUTION CODE: IE22T COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 5 ltr.
 TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc.

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EXTERNAL:	EG&G WILLIAMS, S	4		4	L ST LOBBY WARD	1		1
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Pennsylvania Power & Light Company

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March 14, 1990

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

SUSQUEHANNA STEAM ELECTRIC STATION
LICENSEE EVENT REPORT 90-009-00
FILE R41-2
PLAS - 416

Docket No. 50-387
License No. NPF-14

Attached is Licensee Event Report 90-009-00. This report is being made pursuant to 10CFR50.73(A)(2)(i)(B), in that Susquehanna Unit 1 was in an operating condition prohibited by the Technical Specifications while replacement and testing of a defective pressure switch (Vessel Steam Dome High pressure) was performed. This condition was cleared once the pressure instrument was restored to operable status.

H. G. Stanley
Superintendent of Plant - Susquehanna

HL/mjm

cc: Mr. W. T. Russell
Regional Administrator, Region I
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Mr. G. S. Barber
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LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST 500 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Susquehanna Steam Electric Station - Unit 1							DOCKET NUMBER (2) 0 5 0 0 0 3 8 7			PAGE (3) 1 OF 0 4	
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TITLE (4)
Failed Reactor Pressure Switch - Operation Prohibited by Tech. Spec. During Retest

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)												
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)										
0	2	1	4	9	0	9	0	0	0	9	0	0	0	0	0	0	0	0	0	0	0

OPERATING MODE (9) 1		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)									
POWER LEVEL (10) 1 0 0	20.402(b)	20.406(c)	50.73(a)(2)(iv)	73.71(b)							
	20.406(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)							
	20.406(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)							
	20.406(a)(1)(iii)	X 50.73(a)(2)(i)	50.73(a)(2)(viii)(A)								
	20.406(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)								
20.406(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(ix)									

LICENSEE CONTACT FOR THIS LER (12)

NAME H. Lloyd, Jr. - Power Production Engineer		TELEPHONE NUMBER 7 1 1 7 5 4 2 1 - 3 9 1 1 7	
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS
X	J	C P S	B	0 6 9	Y				

SUPPLEMENTAL REPORT EXPECTED (14)

YES IF YES, COMPLETE EXPECTED SUBMISSION DATE | NO NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On February 14, 1990, with Unit 1 operating at 100% power, it was discovered during surveillance testing that one of four reactor high pressure switches was inoperable. This pressure switch provides a trip function for the Reactor Protection System. The defective instrument was replaced. In order to perform required testing, the affected RPS channel had to be taken out of the tripped condition contrary to the Tech Spec action statement requirement. The cause of this event is attributed to failure of the pressure switch assembly and is considered to be a random failure. This event was determined to be reportable per 10CFR50.73(a)(2)(i)(B), in that a condition prohibited by Tech Specs existed while the channel was removed from the tripped condition. The Technical Specifications contain no provision for allowing an RPS trip signal, imposed by action requirements, to be reset for performance of surveillance tests necessary to restore operability. Such provisions are considered necessary and are being pursued.



LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Unit 1 Susquehanna Steam Electric Station	DOCKET NUMBER (2) 0 5 0 0 0 3 8 7	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		9 0	- 0 0 9	- 0 0	0 2	OF 0 4

TEXT (If more space is required, use additional NRC Form 366A's) (17)

DESCRIPTION OF EVENT

On February 14, 1990, with Unit 1 operating in Condition 1 at 100% power, it was determined during surveillance testing that one of the four reactor steam dome high pressure switches (EIIS Code: I) was inoperable. This pressure switch provides a trip function for the Reactor Protection System (RPS) system. (EIIS Code: JC). The inoperable channel was placed in the trip condition (half scram) as required by Technical Specification 3.3.1 and the applicable Action Statement was entered. This pressure switch was inoperable because repetitive results could not be obtained during the surveillance test.

A replacement pressure switch was bench tested and then installed in place of the defective switch. In order to perform the replacement and subsequent surveillance test, the effected RPS channel had to be reset (i.e. - taken out of the tripped condition). With the channel taken out of the tripped condition, the ACTION statement was no longer met and because Technical Specification does not recognize or allow this action, Technical Specification 3.0.3 was entered. Operation during the time period that the pressure switch was being replaced and tested constituted an operation prohibited by the Technical Specification. This condition existed for one hour and ten minutes.

CAUSE OF EVENT

The cause of this event is attributed to failure of a component or components in the pressure switch assembly. This was determined by evaluation of the surveillance test data which showed that a repeatable as found setpoint could not be obtained.

A replacement pressure switch was bench tested for calibration and time response and was then installed in place of the defective switch. In order to perform the Technical Specification surveillance required to restore operability of the instrument, the half scram, which had been inserted via the 3C manual scram pushbutton as required by the Technical Specification action statement, had to be reset. During the time that the half scram was reset for testing, the action statement was not being met thus constituting a condition prohibited by the Technical Specification.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		9 0	- 0 0 9	- 0 0	0 3	OF 0 4

TEXT (If more space is required, use additional NRC Form 306A's) (17)

REPORTABILITY/ANALYSIS

This event was determined to be reportable per 10CFR50.73(a)(2)(i)(B), in that the Technical Specification 3.3.1 action a., was not met while an RPS instrument was being replaced and tested. The affected RPS trip system had to be reset (i.e. - half scram cleared) to facilitate surveillance testing to restore operability of the instrument. As Technical Specifications does not recognize or allow this action, it constituted operation prohibited by the Technical Specification.

This event did not create a significant degradation in our ability to protect the health and safety of the public and/or plant personnel since only one of two channels in the Div. 1 trip system was inoperable. The other channel in the Div. 1 trip system and both channels of the Div. 2 trip system, as well as all other RPS functions were operable while this condition existed. During the replacement and testing of the defective pressure switch, Div. 2 of RPS reactor high pressure trip was operable as well as all other RPS functions. This event would not have been more significant at any other operating condition.

When the pressure instrument was found to be inoperable, its associated RPS channel was placed in the tripped condition (half scram). Once the instrument was replaced, the trip had to be reset to facilitate testing necessary to ensure it would provide the required trip function. The action statement was not met only during the time period that the defective instrument was being replaced and tested. This time period was one hour and ten minutes.

In accordance with guidance provided in NUREG 1022, Supplement 1, item 14.1; the required submission date for this report was determined to be 03/16/90.

CORRECTIVE ACTION

The defective instrument was replaced and tested satisfactorily. This failure is considered a random failure. The failure was entered into the instrument performance trend program by Instrumentation & Controls personnel (Utility personnel, non-licensed). In addition, this failure was reported to the Nuclear Plant Reliability Data System (NPRDS).

Concerning entry into Limiting Condition for Operation 3.0.3, the Technical Specifications contain no provision for allowing an RPS trip (half scram) signal, imposed due to action requirements, to be reset to allow performance of Technical Specification required surveillances needed to restore the instrument to OPERABLE status. Such provisions are considered necessary and will be pursued as a change to our Technical Specification.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		9 0	- 0 0 9	- 0 0	0 4	OF 0 4

TEXT (If more space is required, use additional NRC Form 308A's) (17)

ADDITIONAL INFORMATION:

Failed Component Identification:

Instrument: Pressure Switch
Model: BIT-M1255-GE
Manufacturer: Barksdale

Previous Similar Events: None