

ACCELERATED DISTRIBUTION DEMONSTRATION SYSTEM

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 9003140414 DOC.DATE: 90/03/05 NOTARIZED: NO DOCKET #
 FACIL: 50-387 Susquehanna Steam Electric Station, Unit 1, Pennsylv 05000387
 AUTH.NAME AUTHOR AFFILIATION
 RYDER, T.S. Pennsylvania Power & Light Co.
 STANLEY, H.G. Pennsylvania Power & Light Co.
 RECIP.NAME RECIPIENT AFFILIATION

SUBJECT: LER 90-003-00: on 900203, reactor coolant conductivity
 sampling frequency not completed within allowances.

W/8 ltr.

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 TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc.

NOTES: LPDR 1 cy Transcripts. 05000387

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	RES/DSIR/EIB		1	1	RGNI FILE 01		1	1
EXTERNAL:	EG&G WILLIAMS, S		4	4	L ST LOBBY WARD		1	1
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Pennsylvania Power & Light Company

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March 5, 1990

U.S. Nuclear Regulatory Commission
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Washington, DC 20555

SUSQUEHANNA STEAM ELECTRIC STATION
LICENSEE EVENT REPORT 90-003-00
FILE R41-2
PLAS -415

Docket No. 50-387
License No. NPF-14

Attached is Licensee Event Report 90-003-00. This report is being made pursuant to 10CFR50.73(a)(2)(i)(B), in that an operation prohibited by the Technical Specifications existed when a surveillance requirement was not completed within the allowances of Technical Specification 4.4.4.



H.G. Stanley
Superintendent of Plant - Susquehanna

TSR/mjm

cc: Mr. W. T. Russell
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
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LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 500 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Susquehanna Steam Electric Station - Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 3 8 7	PAGE (3) 1 OF 0 3
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TITLE (4) **Reactor Coolant Conductivity Sampling Frequency Not Completed Within Allowances of Technical Specification 4.4.4**

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 2	0 3	9 0	9 0	0 0 3	0 0	0 3	0 5	9 0		0 5 0 0 0
										0 5 0 0 0

OPERATING MODE (9) 3	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)									
POWER LEVEL (10) 0 0 0	<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.406(e)	<input type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)						
	<input type="checkbox"/> 20.406(a)(1)(i)	<input type="checkbox"/> 50.36(e)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(e)						
	<input type="checkbox"/> 20.406(a)(1)(ii)	<input type="checkbox"/> 50.36(e)(2)	<input type="checkbox"/> 50.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 368A)						
	<input type="checkbox"/> 20.406(a)(1)(iii)	<input checked="" type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(vii)(A)							
	<input type="checkbox"/> 20.406(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(vii)(B)							
	<input type="checkbox"/> 20.406(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(viii)							

LICENSEE CONTACT FOR THIS LER (12)

NAME T.S. Ryder - Power Production Engineer	TELEPHONE NUMBER AREA CODE: 7 1 1 7 NUMBER: 5 1 4 2 1 - 3 2 3 5
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRPDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRPDS

SUPPLEMENTAL REPORT EXPECTED (14)

<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE) <input checked="" type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15) MONTH: DAY: YEAR:
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ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On February 3, 1990 Unit 1 personnel failed to sample reactor coolant for conductivity within the 4 hour time limit required by Technical Specification 4.4.4.c.1 for plant condition 3. An increased conductivity sampling frequency was required due to the loss of the continuous recording conductivity monitor for reactor coolant when the RWCU System was shut down as part of a planned plant surveillance. Prior to this, RHR Shutdown Cooling had been shutdown as part of the same evolution. With both RWCU and RHR SDC shutdown, there were no available sampling sources for coolant conductivity under existing plant conditions. Due to a power failure, neither RWCU nor RHR SDC could be restored in time to draw a conductivity sample within the prescribed time limit. Upon recovery of RHR SDC, reactor coolant was sampled and conductivity was verified to be within acceptable limits. This event was determined to be reportable per 10CFR50.73(a)(2)(i)(B), in that Unit 1 failed to comply with Technical Specification 4.4.4.c.1 by not being able to meet the increased surveillance activity frequency of sampling for reactor coolant conductivity. There were no safety consequences or compromise to the public health and safety as a result of this incident. The cause of this event is attributed to a planned shutdown of the RWCU and RHR SDC Systems which removed the main and alternate coolant conductivity sampling sources followed by a subsequent power failure preventing restoration of these systems. Without an available sampling source, Technical Specification 4.4.4.c.1 could not be met. The events leading to this occurrence are discussed in detail in LER 90-005.



LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Unit 1 Susquehanna Steam Electric Station	DOCKET NUMBER (2) 0 5 0 0 0 3 8 7	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		9 0	0 0 3	0 0	0 2	OF 0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

DESCRIPTION OF EVENT

At 1632 hours on February 3, 1990 with Unit 1 operating at 0% power in Condition 4, the Reactor Water Cleanup System (RWCU, EIIS Code: CE) was shut down as part of a planned evolution to support Reactor Protection System (RPS, EIIS Code: JC) EPA breaker testing. The shutdown of the RWCU System removed the reactor coolant continuous recording conductivity monitor capability. As a result, an increased surveillance activity for coolant conductivity sampling was effected. Samples were required once every 24 hours in accordance with Technical Specification 4.4.4.c.2 for plant condition 4. With RWCU shutdown, the only other viable source of coolant for conductivity sampling was from the Residual Heat Removal Shutdown Cooling system (RHR SDC, EIIS Code: BO). The RHR SDC System, however, had been previously shutdown at 1555 hours to support the same testing.

At 1725 hours on the same day, the capability of restoring RWCU and RHR SDC was lost due to a power failure as described in LER 90-005. With the inability to restore these systems, no available source remained for drawing a reactor coolant conductivity sample. At 1753 hours, as also described in LER 90-005, the inability to restore RHR SDC resulted in an unplanned entry into plant condition 3. As a consequence, an increased coolant conductivity sampling frequency of once every 4 hours was required in accordance with Technical Specification 4.4.4.c.1.

At 2153 hours, the 4 hour time limit expired without the plant being able to meet the sampling requirements due to the fact there was still no available source for drawing a coolant sample. At 2240 hours during recovery from the LER 90-005 event, RWCU was placed back in service. However, there was insufficient pressure to obtain a reactor coolant sample. At 2302 hours, RHR SDC was restored. At 2330 hours, Chemistry took a coolant sample and verified conductivity to be within acceptable limits.

CAUSE OF EVENT

The cause of this event is attributed to a planned shutdown of the RWCU and RHR SDC Systems which removed the main and alternate coolant conductivity sampling sources followed by a subsequent power failure preventing restoration of these systems. Without an available sampling source, Technical Specification 4.4.4.c.1 could not be met. The events leading to this occurrence are discussed in detail in LER 90-005.

**LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION**

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 60.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-830), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Unit 1 Susquehanna Steam Electric Station	DOCKET NUMBER (2) 0 5 0 0 0 3 8 7 9 1 0 0 0 3 0 1 0 3	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		

TEXT (If more space is required, use additional NRC Form 388A's) (17)

REPORTABILITY/ANALYSIS

This event was determined to be reportable per 10CFR50.73(a) (2) (i) (B), in that Unit 1 failed to comply with Technical Specification 4.4.4.c.1 by not being able to meet the increased surveillance activity frequency of sampling for reactor coolant conductivity. There were no safety consequences or compromise to the public health and safety as a result of this incident. When the surveillance activity was completed 1 hour and 37 minutes after the required time limit, reactor coolant conductivity was verified to be within acceptable limits.

In accordance with the guidance provided in NUREG 1022 Supplement 1 Items 14.1 and 14.10, the required submission date for this report was determined to be March 5, 1990.

CORRECTIVE ACTIONS

After recovery of the RHR SDC System, Chemistry sampled reactor coolant for conductivity and verified the results to be within acceptable limits.

ADDITIONAL INFORMATION

Failed Component Identification: Not applicable

Previous Similar Events: Not applicable

