UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PENNSYLVANIA POWER & LIGHT COMPANY

ALLEGHENY ELECTRIC COOPERATIVE, INC.

DOCKET NO. 50-387

SUSQUEHANNA STEAM ELECTRIC STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 91
License No. NPF-14

1. The Nuclear Regulatory Commission (the Commission or the NRC) having found that:

   A. The application for the amendment filed by the Pennsylvania Power & Light Company, dated February 24, 1989 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-14 is hereby amended to read as follows:

   (2) Technical Specifications and Environmental Protection Plan

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 91 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. PP&L shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance, to be implemented prior to startup, following the Unit 1 fourth refueling and inspection outage, expected to occur on June 2, 1989.

FOR THE NUCLEAR REGULATORY COMMISSION

/s/

Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 22, 1989

PDI-2/LA MO'Brien 5/15/89	PDI-2/PM MThadani:tr 5/12/89	OGC APH 5/14/89	PDI-2/D WButler 5/10/89

3. This license amendment is effective as of its date of issuance, to be implemented prior to startup, following the Unit 1 fourth refueling and inspection outage, expected to occur on June 2, 1989.

FOR THE NUCLEAR REGULATORY COMMISSION

Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 22, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 91

FACILITY OPERATING LICENSE NO. NPF-14

DOCKET NO. 50-387

Replace the following pages of the Appendix A Technical Specifications with enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The overleaf pages are provided to maintain document completeness.*

REMOVE	INSERT
3/4 6-21	3/4 6-21
3/4 6-22*	3/4 6-22*
3/4 6-23*	3/4 6-23*
3/4 6-24	3/4 6-24
3/4 8-29*	3/4 8-29*
3/4 8-30	3/4 8-30

TABLE 3.6.3-1 (Continued)

PRIMARY CONTAINMENT ISOLATION VALVES

VALVE FUNCTION AND NUMBER	MAXIMUM ISOLATION TIME (Seconds)	ISOLATION SIGNAL(s)[a]
Automatic Isolation Valves (Continued)		
Containment Atmosphere Sample		
SV-15734 A,B	N/A	B,Y
SV-15736 A	N/A	B,Y
SV-15736 B	N/A	B,Y
SV-15740 A,B	N/A	B,Y
SV-15742 A,B	N/A	B,Y
SV-15750 A,B	N/A	B,Y
SV-15752 A,B	N/A	B,Y
SV-15774 A,B	N/A	B,Y
SV-15776 A	N/A	B,Y
SV-15776 B	N/A	B,Y
SV-15780 A,B	N/A	B,Y
SV-15782 A,B	N/A	B,Y
Nitrogen Makeup		
SV-15737	N/A	B,Y,R
SV-15738	N/A	B,Y,R
SV-15767	N/A	B,Y,R
SV-15789	N/A	B,Y,R
Reactor Coolant Sample		
HV-143F019	2	B,C
HV-143F020	2	B,C
Liquid Radwaste		
HV-16108 A1,A2	15	B,Z
HV-16116 A1,A2	15	B,Z
RHR - Suppression Pool Cooling/Spray[c]		
HV-151F028 A,B	90	X,Z
CS Test[b][c]		
HV-152F015 A,B	60	X,Z
HPCI Suction[b][c]		
HV-155F042	90	L,LB

TABLE 3.6.3-1 (Continued)

PRIMARY CONTAINMENT ISOLATION VALVES

VALVE FUNCTION AND NUMBER	MAXIMUM ISOLATION TIME (Seconds)	ISOLATION SIGNAL(s)[a]
Automatic Isolation Valves (Continued)		
Suppression Pool Cleanup[b]		
HV-15766	30	A,Z
HV-15768	30	A,Z
HPCI Vacuum Breaker		
HV-155F075	15	LB,Z
HV-155F079	15	LB,Z
RCIC Vacuum Breaker		
HV-149F062	10	KB,Z
HV-149F084	10	KB,Z
TIP Ball Valves [d]		
C51-J004 A,B,C,D,E	5	A,Z
b. Manual Isolation Valves		
MSIV-LCS Bleed Valve		
HV-139F001 B,F,K,P		
Feedwater[e]		
HV-141F032 A,B		
RWCU Return		
HV-14182 A,B		
RCIC Injection		
HV-149F013		
1-49-020		

TABLE 3.6.3-1 (Continued)

PRIMARY CONTAINMENT ISOLATION VALVES

VALVE FUNCTION AND NUMBER
Manual Isolation Valves (Continued)
RCIC Suction[b][c]
HV-149F031
RCIC Turbine Exhaust[b]
HV-149F059
RCIC Vacuum Pump Discharge[b]
HV-149F060
HPCI Injection
HV-155F006
1-55-038
RHR - Shutdown Cooling Return/ LPCI Injection
HV-151F015 A,B
RHR - Suppression Pool Suction[b][c]
HV-151F004 A,B,C,D
RHR Heat Exchanger Vent[c]
HV-151F103 A,B
CS Injection
HV-152F005 A,B
HV-152F037 A,B
CS Suction[b][c]
HV-152F001 A,B
Containment Instrument Gas
SV-12654 A,B

TABLE 3.6.3-1 (Continued)

PRIMARY CONTAINMENT ISOLATION VALVES

VALVE FUNCTION AND NUMBER

Manual Isolation Valves (Continued)

SLCS[b]

HV-148F006

Demineralized Water

1-41-017
1-41-018

ILRT

1-57-193
1-57-194

HPCI Turbine Exhaust[b]

HV-155F066

RHR - Shutdown Cooling Return/
LPCI Injection - Pressure Equalizing Valve

HV-151F122 A,B

c. Other Valves

Feedwater

141F010 A,B

RHR - Shutdown Cooling Suction

PSV-151F126

RHR - Shutdown Cooling Return/
LPCI Injection

HV-151F050 A,B

RHR-Suppression Pool
Cooling/Spray[c]

HV-151F011 A,B

TABLE 3.8.4.2.1-1

MOTOR OPERATED VALVES THERMAL OVERLOAD PROTECTION - CONTINUOUS

VALVE NUMBER	SYSTEM(S) AFFECTED
HV-01222A	RHRSW
HV-01222B	RHRSW
HV-01224A1	RHRSW
HV-01224B1	RHRSW
HV-01224A2	RHRSW
HV-01224B2	RHRSW
*HV-01112A	ESW
*HV-01112B	ESW
*HV-01122A	ESW
*HV-01122B	ESW
*HV-01112C	ESW
*HV-01112D	ESW
*HV-01122C	ESW
*HV-01122D	ESW
*HV-01110A	ESW
*HV-01110B	ESW
*HV-01120A	ESW
*HV-01120B	ESW
*HV-01110C	ESW
*HV-01110D	ESW
*HV-01120C	ESW
*HV-01120D	ESW
*HV-01110E	ESW
*HV-01120E	ESW
*HV-01112E	ESW
*HV-01122E	ESW
HV-08693A	ESW
HV-08693B	ESW
HV-01201A1	RHRSW
HV-01201A2	RHRSW
HV-01201B1	RHRSW
HV-01201B2	RHRSW
HV-11210A	RHRSW
HV-11210B	RHRSW
HV-11215A	RHRSW
HV-11215B	RHRSW
HV-15766	Cont. Isol.
HV-15768	Cont. Isol.
HV-12603	Cont. Isol.
HV-11345	Cont. Isol.
HV-11313	Cont. Isol.
HV-11346	Cont. Isol.
HV-11314	Cont. Isol.
HV-E11-1F009	RHR

* Continuous bypass not required when corresponding diesel generator is not aligned to the Class 1E distribution system.

TABLE 3.8.4.2.1-1 (Continued)

MOTOR OPERATED VALVES THERMAL OVERLOAD PROTECTION CONTINUOUS

VALVE NUMBER	SYSTEM(S) AFFECTED
HV-E11-1F040	RHR
HV-G33-1F001	RWCU
HV-E11-1F103A	RHR
HV-E11-1F075A	RHRSW
HV-E11-1F048A	RHR
HV-E11-1F006C	RHR
HV-E11-1F004C	RHR
HV-E11-1F015A	RHR
HV-E11-1F024A	RHR
HV-E21-1F015A	CS
HV-E41-1F002	HPCI
HV-B21-1F016	NSSS
HV-E11-1F022	RHR
HV-E11-1F010A	RHR
HV-E11-1F004A	RHR
HV-E11-1F006A	RHR
HV-E11-1F027A	RHR
HV-E11-1F007A	RHR
HV-E11-1F104A	RHR
HV-E11-1F028A	RHR
HV-E11-1F047A	RHR
HV-E11-1F073A	RHRSW
HV-E11-1F003A	RHR
HV-E11-1F017A	RHR
HV-E21-1F001A	CS
HV-E21-1F031A	CS
HV-E21-1F004A	CS
HV-E21-1F005A	CS
HV-E11-1F021A	RHR
HV-E11-1F016A	RHR
HV-15112	RHR
HV-E51-1F007	RCIC
HV-E51-1F084	RCIC
HV-E11-1F027B	RHR
HV-E11-1F048B	RHR
HV-E11-1F015B	RHR
HV-E11-1F006B	RHR
HV-E11-1F021B	RHR
HV-E11-1F010B	RHR
HV-E11-1F004B	RHR
HV-E11-1F007B	RHR
HV-E11-1F104B	RHR