

# ACCELERATED DISTRIBUTION DEMONSTRATION SYSTEM

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SESSION NBR: 8810060054 DOC. DATE: 88/09/29 NOTARIZED: NO DOCKET #  
CIL: 50-387 Susquehanna Steam Electric Station, Unit 1, Pennsylvania 05000287  
50-388 Susquehanna Steam Electric Station, Unit 2, Pennsylvania 05000388  
AUTH. NAME AUTHOR AFFILIATION  
KEISER, H.W. Pennsylvania Power & Light Co.  
RECIP. NAME RECIPIENT AFFILIATION  
BELLAMY, R.R. Region 1, Ofc of the Director

SUBJECT: Forwards util evaluation of incumbent radiological operations supervisor qualifications to meet Tech Specs. R

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NOTES: LPDR 2 cys Transcripts. 50-387/87-19 05000387 S  
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**Pennsylvania Power & Light Company**

Two North Ninth Street • Allentown, PA 18101 • 215/770-5151

SEP 29 1988

Harold W. Keiser  
Senior Vice President-Nuclear  
215/770-4194

Mr. Ronald R. Bellamy  
Chief Facilities Radiological Safety and Safeguards Branch  
Division of Radiation Safety and Safeguards  
U.S. Nuclear Regulatory Commission  
Region I  
475 Allendale Road  
King of Prussia, PA 19406

SUSQUEHANNA STEAM ELECTRIC STATION  
NRC INSPECTION REPORTS 50-387/87-19  
AND 50-388/87-19  
PLA-3091 FILE R41-1C, R41-2

Docket Nos. 50-387  
and 50-388

Dear Mr. Bellamy:

In response to your letter of July 15, 1988, attached is the Pennsylvania Power & Light Company (PP&L) evaluation of the incumbent Radiological Operations Supervisor's qualifications to meet Technical Specification requirements.

Very truly yours,

H. W. Keiser

Attachment

cc: NRC Document Control Desk (original)  
NRC Region I  
Mr. F. I. Young, NRC Resident Inspector  
Mr. M. C. Thadani, NRC Project Manager

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INCUMBENT RADIOLOGICAL OPERATIONS SUPERVISOR QUALIFICATIONS

The incumbent will possess a minimum of four years of experience in the discipline he supervises in January 1989. At this time, he will have a full two years in his current position. The remaining two years of experience is credited as follows:

1. One year credit for HP technician duties which were performed while the incumbent was a Licensed Reactor Operator and graduate student at a University research facility. As an operator and graduate student he performed activities which included external radiation surveys, contamination surveys, gamma spectroscopy, etc. One year equivalent Health Physics experience was credited for his approximately 1 year 9 months research reactor experience.
2. One year credit for the incumbents five year assignment with the Division of Naval Reactors. Almost four years of this work experience was in the area of surface ship refueling. In this capacity he was responsible for the safety and quality of refueling work for 12 reactor plants. His duties included the design, development, review and approval of refueling concepts, equipment, casks, shielding and procedures. This credit conservatively applies the weighting factors (Table 1) experience requirements specified in ANSI N18.1 and ANSI/ANS-3.1-1987. PP&L also considers this credit to be equitable and acceptable for the ANSI 18.1-1971 version.

In addition, the incumbent has over 20 years experience in design and supervisory aspects of operating nuclear power facilities. His depth of knowledge, technical, supervisory and administrative skills were the prime consideration in his appointment to his present position.

As further assurance that the incumbent has received training appropriate and sufficient for the position, an individualized training program has been prepared for this subject individual. The outline for this supplemental training program is attached. Upon completion of this training or January 1, 1989, whichever is latest, the subject individual shall be considered to be fully qualified to the requirement of the Susquehanna SES Technical Specifications.

RDK:tah  
rdkmeilla



Supplemental Training Program  
Radiological Operations Supervisor

The attached outline lists those documents which must be read during the supplemental training period by the Radiological Operations Supervisor. A signature on the required reading list indicates that the trainee has demonstrated familiarity with the documents contents to the Chemistry/Health Physics Supervisor or his designee.

In addition to satisfactorily completing the required reading list, the Radiological Operations Supervisor shall attend workshops on Radioactive Waste Packaging, Transportation and Disposal, and on Regulations and the Regulatory process. The Radioactive Waste Workshop will be the one presented by Chem Nuclear Systems, Inc. or equivalent. The Regulatory Workshop will be presented by a Board Certified Health Physicist with more than 20 years of regulatory experience.

This training program shall be completed prior to December 31, 1988.

Prepared by:

George H. Smith 6/6/88  
George H. Smith, CHP

Approved by:

Harry L. Riley 6-6-88  
Harry L. Riley  
Chemistry/Health Physics Supv.

Trainee Signature      Date

Federal Regulations

- |                          |   |       |       |
|--------------------------|---|-------|-------|
| 10 CFR 2                 | Appendix C Rules of Practice<br>for Domestic Licensing<br>Proceedings   | _____ | _____ |
| 10 CFR 19                | Notices, Instructions &<br>Reports  | _____ | _____ |
| 10 CFR 20                | Standards for Protection<br>Against Radiation   | _____ | _____ |
| 10 CFR 30                | Rules of General Applicability<br>to Domestic Licensing of<br>By-product Material   | _____ | _____ |
| 10 CFR 40                | Domestic Licensing of Source<br>Material  | _____ | _____ |
| 10 CFR 50                | Domestic Licensing of<br>Production and Utilization<br>Facilities   | _____ | _____ |
| 10 CFR, Part 61,         | Management and Disposal<br>of Low Level Waste by Shallow Land<br>Burial and Alternative Disposal                                | _____ | _____ |
| 10 CFR 70                | Domestic Licensing of Special<br>Nuclear Material   | _____ | _____ |
| 10 CFR 71                | Packaging of Radioactive<br>Material for Transport and<br>Transportation of Radioactive<br>Material Under Certain<br>Conditions | _____ | _____ |
| 10 CFR 150               | Exemptions & Continued<br>Regulatory Authority in<br>Agreement States & in Offshore<br>Waters Under Section 274                 | _____ | _____ |
| 29 CFR 1910              | General Industry Safety &<br>Health Standards   | _____ | _____ |
| 30 CFR 11                | Respiratory Protective Devices,<br>Tests for Permissibility, Fees   | _____ | _____ |
| 49 CFR, Parts 100 - 199, | Transportation  | _____ | _____ |





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Trainee Signature

Date

Burial Site Regulations

Barnwell, South Carolina  
Disposal Site Criteria

Chem-Nuclear Systems, Inc.  
South Carolina Operating License  
for Barnwell Disposal Site,  
License No. 097

Chem-Nuclear Systems, Inc., NRC  
Material License 46-13536-01

US Ecology Beatty Disposal Site  
state of Nevada Radioactive  
Material License 13-11-0043-02

Site disposal criteria for the  
US Ecology Richland, Washington  
Disposal Site, Radioactive Material  
License WN-1019-2

Technical Specifications

Section 6.0 Administrative Controls

FSAR

Section 11.4 Solid Radwaste Management  
System

Section 12.5 Health Physics Program

Regulatory Guide

Reg Guide 1.8 Personnel Selection &  
Training

Reg Guide 1.16 Reporting of Operating  
Information-Appendix A  
Technical Specifications

Reg Guide 1.21 Measuring, Evaluating and  
Reporting Radioactive  
Material in Solid Wastes  
and Release of Radioactive  
Material in Liquid and  
Gaseous Effluents from  
Light Water Cooled Nuclear  
Power Plants



		<u>Trainee Signature</u>	<u>Date</u>
Reg Guide 1.33	Quality Assurance Program Requirements	_____	_____
Reg Guide 1.88	Collection, Storage & Maintenance of Nuclear Power Plant Quality Assurance Records	_____	_____
Reg Guide 7.1	Administrative Guide for Packaging and Transporting Radioactive Material	_____	_____
Reg Guide 7.10	Establishing Quality Assurance Programs for Packaging used in the Transport of Radioactive Material	_____	_____
Reg Guide 8.4	Direct Reading and Indirect Reading Pocket Dosimeters	_____	_____
Reg Guide 8.8	Information Relevant to Ensuring Occupational Radiation Exposures at Nuclear Power Stations will be ALARA	_____	_____
Reg Guide 8.9	Acceptable Concepts, Models, Equations Assumptions for a Bioassay Program	_____	_____
Reg Guide 8.10	Operating Philosophy for Maintaining Occupational Radiation Exposures ALARA	_____	_____
Reg Guide 8.13	Instruction Concerning Prenatal Radiation Exposure	_____	_____
Reg Guide 8.14	Personnel Neutron Dosimeters	_____	_____
Reg Guide 8.15	Acceptable Programs for Respiratory Protection	_____	_____
Reg Guide 8.19	Occupational Radiation Dose Assessment in Light-Water Reactor Power Plants Design Stage Man-Rem Estimates	_____	_____
Reg Guide 8.25	Calibration and Error Limits of Air Sampling Instruments for Total Volume of Air Sampled	_____	_____



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	<u>Trainee Signature</u>	<u>Date</u>
Reg Guide-8.26 Application of Bioassay for Fission & Activation Products	_____	_____
Reg Guide 8.28 Audible Alarm Dosimeters	_____	_____
Reg Guide 8.29 Instruction Concerning Risk from Occupational Radiation Exposure	_____	_____
Reg Guide 10.1 Compilation of Reporting Requirements for Persons Subject to NRC Regulations	_____	_____

ANSI Documents

ANSI N18.1-1971 Selection & Training of Nuclear Power Plant Personnel	_____	_____
ANSI N323.1978 American National Standard Radiation Protection Instrumentation Test and Calibration	_____	_____
ANSI N13.27.1978 American National Standard Performance Requirements for Pocket-sized Alarm Dosimeters and Alarm Ratemeters	_____	_____
ANSI N13.5.1972 American National Standard Performance Specifications for Direct Reading and Indirect Reading Pocket Radiation	_____	_____
ANSI 13.4.1971 American National Standard for the Specification of Portable X-or Gamma Radiation Survey Instruments	_____	_____
ANSI N42.12.1980 American National Standard Calibration and Usage of Sodium Iodide Detector Systems	_____	_____
ANSI 42.14-1978 American National Standard Calibration and Usage of Germanium Detectors for Measurement of Gamma-ray Emission of Radionuclides	_____	_____

		<u>Trainee Signature</u>	<u>Date</u>
ANSI N343-1978	American National Standard. for Internal Dosimetry for Mixed Fission and Activation Products	_____	_____
ANSI Z88.2-1980	American National Standard Practices for Respiratory Protection	_____	_____
ANSI/ASME N510-1980	American National Standard Testing of Nuclear Air- Cleaning Systems	_____	_____
ANSI N13.1	Guide to Sampling Airborne Radioactive Materials in Nuclear Facilities	_____	_____
ANSI/ANS-3.1- 1987	Selection, Qualification & Training of Personnel for Nuclear Power Plants	_____	_____
Draft Std. N13.12	Surface Radioactivity Guide for Materials, Equipment & Facilities to be Released for Uncontrolled Use	_____	_____
Draft Std. N13.30	Performance Criteria for Bioassay	_____	_____

Significant NRC Bulletins, Circular & Information Notices - Generic Letters

Bulletin 79.19:	Packaging of Low Level Radioactive Waste for Transport & Burial	_____	_____
Circular 80.18:	10 CFR 50.59 Safety Evaluation for Changes to Radioactive Waste Treatment Systems	_____	_____
Information Notice 81-26:	Part 3, Placement of Personnel Monitoring Devices for External Radiation Exposure (8/28/81) and Supplement 1 (7/19/82)	_____	_____
Information Notice 83-59:	Dose Assignment for Workers in Nonuniform Radiation Fields	_____	_____
Circular 81-07:	Control of Radioactivity Contaminated Material	_____	_____



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Information Notice 85-92:	Surveys of Wastes Before Disposal from Nuclear Reactor Facilities	_____	_____
Information Notice 86-23:	Excessive Skin Exposures Due to Contamination with Hot Particles	_____	_____
Bulletin 80-10:	Contamination of Nonradioactive System and Resulting Potential for Unmonitored, Uncontrolled Release of Radioactivity to Environment	_____	_____
Information Notice 86-44:	Failure to Follow Procedure When Working in High Radiation Areas	_____	_____
Information Notice 86-20:	Low Level Radioactive Waste Scaling Factors, 10 CFR 61	_____	_____
Information Notice 82-18:	Assessment of Intake of Radioactive Material by Workers	_____	_____
Information Notice 86-43:	Problems With Silver Zeolite Sampling of Airborne Radioiodine	_____	_____
Information Notice 82-49:	Correction for Sample Conditions for Air and Gas Monitoring	_____	_____
Generic Letter 81-38	Storage of Low Level Waste at Power Reactor Sites and Enclosure Radiological Safety Guidance for Onsite Contingency Storage Capacity	_____	_____
Generic Letter 85-14	Commercial Storage at Power Reactor Sites of Low Level Radioactive Waste Not Generated by the Utility	_____	_____
<u>NUREG's</u>			
NUREG 0041	Manual of Respiratory Protection Against Airborne Radioactive Material	_____	_____

		<u>Trainee Signature</u>	<u>Date</u>
NUREG 0855	Health Physics Appraisal Program	_____	_____
NUREG/CR-0466	Determining Effectiveness of ALARA Design and Operational Features	_____	_____
NUREG/CR-3544	Beta Particle Measurement and Dosimetry at NRC Licensed Facilities	_____	_____
NUREG/CR-2956	Neutron Dosimetry at Commercial Nuclear Plants	_____	_____
<u>INPO</u>			
INPO 85-004	Guidelines for the Conduct of Radiological Protection at Nuclear Power Stations (Preliminary)	_____	_____
INPO 82-001 (OEN-01)	Strippable Decontamination Coating	_____	_____
INPO 82-001 (OEN-02)	Reactor Vessel Head Shield	_____	_____
INPO 82-001 (OEN-08A)	ALARA Planning for Station Work	_____	_____
INPO 82-001 (OEN-09)	Low-Level Radioactive Waste Management	_____	_____
INPO 82-001 (EPN-02)	Conduct of a Direct-Reading Dosimeter Program (Quartz Fiber Pocket Dosimeters)	_____	_____
INPO 82-001 (EPN-03)	Comparison of Dosimetry Results	_____	_____
INPO 82-001 (FDO-001)	Respirator Cleaning and Maintenance Packages	_____	_____
INPO 82-001 (FDO-02)	Modular Contamination Control Enclosures	_____	_____
INPO 86-011 (RP-601)	Use of Goals in Reducing Personnel Radiation Doses	_____	_____



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RP-603

Advanced Worker Training  
in Radiological Protection  
(Preliminary)

Trainee Signature

Date

INPO-88-004  
(RP-604)

Monitoring Personnel for  
Radioactive Contamination

Reports

Swedish Experience in  
Reducing Occupational  
Radiation Exposure

PP&L Procedure Reading Requirements

NDI 6.4.1

Susquehanna Radiation  
Protection Program

NDI 6.4.2

ALARA Policy Program

NDI 6.4.3

Personnel Radiation  
Exposure Policy

NDI 6.4.4

Prenatal Radiation  
Exposure Policy

NDI 6.4.5

Respiration Protection  
Program

NDI 6.4.8

Personnel External  
Dosimetry Program  
Responsibilities &  
Interfaces

NDI-QA-6.5.1

Radwaste Program

NDI-QA-8.1.1

NQA Section Charter

AD-00-700

Conduct of Health Physics

AD-00-705

Access Control & RWP System

AD-00-710

Radiation Survey Program

AD-00-715

Airborne Radioactive  
Material Concentration  
Sampling & Evaluation  
Program

AD-00-720

Contamination Control



		<u>Trainee Signature</u>	<u>Date</u>
AD-00-725	Respiratory Protection Program	_____	_____
AD-00-730	Health Physics Training Programs	_____	_____
AD-00-735	External Dosimetry Program	_____	_____
AD-00-740	Internal Dosimetry Program	_____	_____
AD-00-745	ALARA Program.	_____	_____
AD-00-750	Control of Sources	_____	_____
AD-00-760	Performance Evaluation Program & Administrative Controls for Health Physics Instrumentation	_____	_____
AD-00-765	Solid Radwaste Program	_____	_____
AD-00-770	Operating of Low Level Radwaste Holding Facility	_____	_____
AD-00-775	Decontamination Program	_____	_____
AD-QA-502	Work Authorization System	_____	_____
AD-QA-410	Plant Modification Program	_____	_____
AD-QA-111	Radwaste Management Program	_____	_____
AD-QA-305	Radwaste Process	_____	_____
AD-QA-112	Radwaste Volume Reduction Committee	_____	_____
AD-QA-311	Radwaste Process Control	_____	_____
AD-QA-502	Work Authorization System	_____	_____
HP-TP-807	Specific Site Criteria for Radwaste Shipment to the Chem-Nuclear Systems, Inc. Barnwell, South Carolina Disposal Site	_____	_____
HP-TP-808	Specific Site Criteria for Radioactive Material Shipments to the US Ecology Beatty Nevada Disposal Site	_____	_____



		<u>Trainee Signature</u>	<u>Date</u>
HP-TP-809	Specific Site Criteria for Radioactive Material Shipments to the US Ecology Richland, Washington Disposal Site	_____	_____
HP-AL-400	RWP/ALARA Reviews & Evaluations	_____	_____
HP-TP-320	Radiation Work Permits (RWPS)	_____	_____
HP-HI-009	Qualification for HP Tech. (Level II) to Prepare/ Approve Radiation Work Permits	_____	_____



