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SUBJECT: Application for amend to License NPF-14 changing Tech Specs reactor vessel matl surveillance withdrawal schedule.

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JUN 0 3 1988

Director of Nuclear Reactor Regulation
Attention: Dr. W.R. Butler, Project Director
Project Directorate I-2
Division of Reactor Projects
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

SUSQUEHANNA STEAM ELECTRIC STATION
PROPOSED AMENDMENT 112 TO LICENSE NO. NPF-14;
CHANGES TO THE SSES UNIT 1
TECHNICAL SPECIFICATIONS TO REVISE THE SURVEILLANCE
CAPSULE WITHDRAWAL SCHEDULE
PLA-3034 FILES A17-2, R41-2 Docket No. 50-387

Dear Dr. Butler:

Background

The purpose of this letter is to propose a change to the Susquehanna Steam Electric Station Unit 1 Technical Specification requirement for the Reactor Vessel Material Surveillance Withdrawal Schedule and also to revise the Material Surveillance Lead Factor Ratio. The revision to the withdrawal schedule complies with the latest revision of 10 CFR50, Appendix H as well as ASTM Standard E-185-82, "Conducting Surveillance Tests for Light-Water Cooled Nuclear Power Reactor Vessels, E706." The lead factor ratio has been revised in accordance with requirements of General Electric letter GP-83-213, "Appendices G and H Question Responses." These changes are consistent with the Technical Specifications for Unit 2.

Description of Change

PP&L is proposing that Technical Specification, Table 4.4.6.1.3-1, Reactor Vessel Material Surveillance Program-Withdrawal Schedule be revised. Attached is a marked-up copy of this table. Two changes to the table are a revised lead factor and also a withdrawal time schedule based on effective full power years (EFPY).

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Safety Analysis

The Reactor Vessel Material Surveillance Program-Withdrawal Schedule, has been revised to conform to 10 CFR 50, Appendix H, Reactor Vessel Material Surveillance Program Requirements. Instead of using annual years for the scheduled withdrawals Appendix H references the ASTM Standard E-185-82 to the extent practical for this withdrawal schedule. Effective full power years are now used instead of annual years. The effective full power year schedule for a life of 32 years is as follows:

Capsule 1 is to be withdrawn at 6 EFPY,
Capsule 2 at 15 EFPY,
Capsule 3 which is a spare at the end of plant life.

Previously the life of the plant was considered 40 years. Based on this the first capsule was to be withdrawn at 10 years, the second at 20 years and the third at the end of life or 40 years.

In addition, General Electric reviewed the lead factor calculations and determined that the lead factor for all three capsules was 1.20 instead of 0.6. This calculation complies with the ASTM Standard and is the ratio of the neutron flux density at the surveillance sample divided by the neutron flux density at the one quarter thickness of the vessel. The 1.2 ratio falls between 1 and 3 as listed in ASTM E-185-82.

No Significant Hazards Considerations

The proposed changes do not:

I. Involve a significant increase in the probability or consequences of an accident previously evaluated. Both the revised lead factor and surveillance capsule withdrawal schedule are used only to monitor station operation and do not impact on station accidents. Also, the location of the capsules and analyses performed on them will remain the same with this proposed Tech Spec change.

II. Create the possibility of a new or different kind of accident from any accident previously evaluated. These changes only affect withdrawal schedule and the lead factor and do not propose any physical changes to the reactor vessel wall. Also, the location of the capsules and attachment to the vessel wall will remain the same with this Tech Spec change.

III. Reduce the margin of plant safety. The lead factor now falls within the acceptable range stated in ASTM E-185-82 and the revised capsule withdrawal schedule is more conservative then the original schedule. Based on this, no significant reduction in any safety margin will occur due to this change.

Any questions on the above material should be directed to Mr. J. S. Fields, at (215) 770-7911. Pursuant to 10 CFR 170, the appropriate fee is enclosed.

Very truly yours,

H. W. Keiser

Attachment

cc: NRC Document Control Desk (original)
NRC Region I
Mr. F. I. Young, NRC Sr. Resident Inspector-SSES
Mr. M. C. Thadani, NRC Project Manager
Mr. T. M. Gerusky, Pa. DER