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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
REGION III  
799 ROOSEVELT ROAD  
GLEN ELLYN, ILLINOIS 60137

MAR 26 1976

Commonwealth Edison Company  
ATTN: Mr. Byron Lee, Jr.  
Vice President  
P. O. Box 767  
Chicago, Illinois 60690

Docket No. 50-237

Gentlemen:

This refers to the inspection conducted by Mr. J. E. Kohler of this office on March 2-3, 1976, of activities at Dresden Nuclear Power Station, Unit 2, authorized by NRC Reactor Operating License No. DPR-19 and to the discussion of our findings with Mr. Abel at the conclusion of the inspection.

The enclosed copy of our inspection report identifies areas examined during the inspection. Within these areas, the inspection consisted of a selective examination of procedures and representative records, observations, and interviews with personnel.

No items of noncompliance with NRC requirements were identified within the scope of this inspection.

In accordance with Section 2.790 of the NRC's "Rules of Practice," Part 2, Title 10, Code of Federal Regulations, a copy of this letter and the enclosed inspection report will be placed in the NRC's Public Document Room, except as follows. If this report contains information that you or your contractors believe to be proprietary, you must apply in writing to this office, within twenty days of your receipt of this letter, to withhold such information from public disclosure. The application must include a full statement of the reasons for which the information is considered proprietary, and should be prepared so that proprietary information identified in the application is contained in an enclosure to the application.

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Commonwealth Edison  
Company

- 2 -

MAR 26 1976

We will gladly discuss any questions you have concerning this inspection.

Sincerely yours,

Gaston Fiorelli, Chief  
Reactor Operations and  
Nuclear Support Branch

Enclosure:  
IE Inspection Report  
No. 050-237/76-04

cc w/encl:  
Mr. B. Stephenson, Station  
Superintendent

bcc w/encl:  
PDR  
Local PDR  
NSIC  
TIC  
Anthony, Roisman, Esq.,  
Attorney

UNITED STATES NUCLEAR REGULATORY COMMISSION  
OFFICE OF INSPECTION AND ENFORCEMENT

REGION III

Report of Operations Inspection

IE Inspection Report No. 050-237/76-04

Licensee: Commonwealth Edison Company  
P. O. Box 767  
Chicago, Illinois 60690

Dresden Nuclear Power Station  
Unit 2  
Morris, Illinois

License No. DPR-19  
Category: C

Type of Licensee: BWR (GE) 810 Mwe

Type of Inspection: Routine, Announced

Dates of Inspection: March 2 and 3, 1976

Principal Inspector: *J. E. Kohler*  
J. E. Kohler

                      
(Date)

Accompanying Inspectors: None

Other Accompanying Personnel: None

Reviewed By: *W. S. Little*  
W. S. Little, Chief  
Nuclear Support Section

3/25/76  
(Date)

## SUMMARY OF FINDINGS

### Inspection Summary

The inspection on March 2-3, (76-04): consisted of review of procedures applicable to the March 1976 refueling and followup on previous noncompliance items.

### Enforcement Items

None.

### Other Significant Items

#### A. Systems and Components

Unresolved Item - A possible loss of secondary containment on loss of station power occurred when the "A" SGTS and the number 3 diesel generator were coincidentally out of service on March 3, 1976. This item will be followed up in a subsequent inspection. The licensee reported this to IE:III on March 3, 1976.

#### B. Facility Items (Plans and Procedures)

- (1) The 1976 refueling outage for Dresden 2 has slipped from March 7 to March 13, 1976.
- (2) During the 1976 refueling outage, 160 new 8x8 assemblies will be inserted in the Unit 2 core. This will yield a final core configuration of 284 8x8 assemblies and 440 7x7 assemblies.

#### C. Managerial Items

None.

#### D. Noncompliance Identified and Corrected by Licensee

None.

#### E. Deviations

None.

#### F. Status of Previously Reported Unresolved Items

None.

### Management Interview

A management interview was conducted by Mr. Kohler with Mr. Abel, Administrative Assistant, and other members at the Dresden Station. The following items were discussed.

- A. The inspector discussed the events which took place on March 2 and 3, 1976, that lead up to a reportable occurrence on March 3, 1976, dealing with the possible loss of secondary containment on loss of station power. (Paragraph 2, Report Details)
- B. The inspector stated that he had reviewed the preliminary procedures relating to the Unit 2 refueling outage scheduled for March 13, 1976, and had no comments. (Paragraph 4, Report Details)
- C. The inspector stated that he had selected three safety related maintenance activities to be scheduled during the March 1976 outage for procedure review and found no deficiencies. (Paragraph 3, Report Details)
- D. The inspector stated that he had closed out one outstanding item relating to an infraction identified in IE Inspection Report No. 050-237/75-22. (Paragraph 5, Report Details)

## REPORT DETAILS

### 1. Persons Contacted

J. Abel, Administrative Assistant  
T. Watts, Technical Staff Supervisor  
C. Sargent, Unit 3, Leading Engineer  
J. Kolanowski, Unit 2, Leading Engineer  
J. Dolter, Leading Nuclear Engineer

### 2. Reportable Occurrence

The "A" train of the Standby Gas Treatment System (SGTS) was tagged and taken out of service on March 1, 1976, in preparation for a routine surveillance test. This left the "B" train of the SGTS as the redundant train which would provide the necessary vacuum on the reactor building on loss of station power in order to maintain secondary containment. On loss of power to the station, the "B" SGTS is supplied by the number 3 diesel generator.

At 0615 hours on March 3, 1976, during routine diesel generator surveillance, the number 3 diesel was found to be capable of generating only 1.4 MWe out of a maximum of 2.8 MWe. On March 3, 1976, the number 3 diesel was then taken out of service to investigate the reduction in generator output, and was returned to service when the cause in reduction of generator output was discovered to be a loose governor wire.

A reportable occurrence dealing with possible loss of secondary containment on loss of station power from 0615 to 0815 on March 3, 1976, occurred when the "A" SGTS was out of service coincident with the number 3 diesel generator.

### 3. Safety Related Maintenance

The inspector reviewed the March 1976 outage schedule and selected three major maintenance activities for procedure review. The following maintenance activities selected for review were found to have written procedures which were approved or in the process of approval.

- a. Feedwater nozzle, sparger inspection.
- b. Control rod drive maintenance.

- c. Local leak rate test on main steam line isolation valves.
- d. Inservice Inspection.

4. Refueling Procedures

The following refueling procedures were reviewed by the inspector without comment.

- DFP 800-1 Master Refueling Procedure.
- DFP 800-7 Moving Fuel Inside the Reactor.
- DFP 800-9 Fuel Movement from Spent Fuel Storage Pool to the Reactor.
- DFP 800-10 Fuel Movement from Reactor to Spent Fuel Pit.
- DFP 800-53 Refueling Prerequisites Checksheet.
- DTP 8474 Final Core Verification.
- DTP 8471 Nuclear Material Transfer Checksheets.
- DTP 8472 Bundle Loading and Unloading Sheets.
- DAP 7-7 Administrative Guidance for Personnel Involved in Refueling.

5. Outstanding Item

A previous inspection report (050-237/75-22) identified an infraction pertaining to exceeding the specified two year elapsed time between successive Type B and C containment leak rate tests for Dresden Unit 2. The licensee's corrective action is described in his October 28, 1975, response to our inspection report, and consists of revising the surveillance schedules on containment leak rate testing to specify a two year testing frequency.

The inspector obtained verification that the Type B and C testing frequency was revised on the master surveillance log to specify a test frequency not to exceed two years. This item is considered closed out.