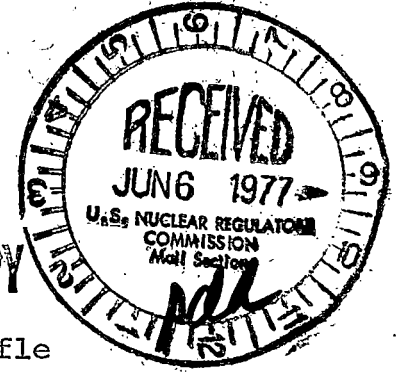




Commonwealth Edison
One First National Plaza, Chicago, Illinois
Address Reply to: Post Office Box 767
Chicago, Illinois 60690

May 27, 1977

Mr. Donald K. Davis, Acting Chief
Operating Reactors - Branch 2
Division of Operating Reactors
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555



REGULATORY DOCKET FILE COPY

Subject: Dresden Station Unit 2 Torus Baffle
Removal; Dresden Station Units 2 & 3,
Quad Cities Station Units 1 & 2 Safety
Relief Valve Installation; Dresden
Station Units 2 & 3, Quad Cities
Station Units 1 & 2 Relief Valve Dis-
charge Line Restraints Installation;
NRC Docket Nos. 50-237, 50-249,
50-254, and 50-265

Reference (a): J. S. Abel letter to D. L. Ziemann dated
December 26, 1974, NRC Docket Nos.
50-237/249 and 50-254/265

Reference (b): Dresden Station Special Report No. 42
Quad Cities Station Special Report No. 17
NRC Docket Nos. 50-237/249 and 50-254/265

Reference (c): G. A. Abrell letter to D. L. Ziemann
dated December 8, 1975, NRC Docket No.
50-237

Dear Mr. Davis:

This letter is to inform you of changes in schedule
for the subject installations and modifications.

Reference (a) indicated our original schedule for
installation of the safety valve premature actuation corrective
measures, which was subsequently amended by Reference (c).
Our current plan is to delay installation of the safety/relief
valves to a refueling outage which will allow the additional

77158023

Mr. Donald K. Davis

-2-

May 27, 1977

outage length to be minimized. This may be achievable during the refueling outages when the Mark I Containment "long term program" modifications are installed. These outages are currently estimated to be 1979-1980. Our reasons for postponing the installations are as follows: (1) Current economic evaluations indicate that it is more cost effective to maintain margin to safety valve setpoints by accepting the required small unit deratings late in the cycle; (2) By combining these installations with other work in the torus (Mark I fixes), additional torus recoatings can be eliminated; (3) New calculational techniques and investigations under way as part of the Mark I Containment evaluations could change calculated relief valve discharge pipe loadings and support requirements in the torus.

Operating with the existing safety valves has been justified by the interim measures to reduce the likelihood of premature valve actuation discussed in Special Report No. 18, Supplement B dated September 25, 1973.

Reference (b) committed the upgrading of electromatic relief valve discharge line restraints during the installation of the safety/relief valves; therefore, these modifications will be delayed and installed during the Mark I Containment fix installations estimated to occur in 1979-1980. Reasons for postponement of these installations would be the same as items 2 and 3 mentioned above for the safety/relief valve installations.

Until the new restraints are installed, the existing relief valve restraints in the torus will be inspected at each scheduled refueling outage in order to ensure continued integrity. This visual inspection will be adequate in view of the very few number of actuations which normally occur during each operating cycle, and in view of the history of satisfactory inspection results.

Reference (c) indicated our plan to remove the Dresden Station Unit No. 2 torus baffles during the 1977 refueling outage. It is advantageous, in order to minimize outage duration, to reschedule the baffle removal to coincide with a refueling outage of extended duration for other reasons. The earliest extended refueling outage is expected to occur in 1979-1980 for installation of "long term containment" modifications.

Commonwealth Edison

Mr. Donald K. Davis

-3-

May 27, 1977

Continued operation with the baffles is justified, because the baffles have been inspected at each refueling outage and found completely intact and without any signs of damage. These inspections will continue at each scheduled refueling outage. The evaluations for operation with the baffles, provided in our letter to you dated November 27, 1972 and the September 9, 1976 letter from L. J. Sobon (GE) to V. Stello (NRC), "Mark I Short Term Program Report Questions" specifically, question CSB-9, are still applicable.

You will be advised of any additional changes to these installation schedules.

Please direct any additional questions to this office.

One (1) signed original and 39 copies are provided for your use.

Very truly yours,



M. S. Turbak
Nuclear Licensing Administrator
Boiling Water Reactors

6 7 3 00

RECEIVED DOCUMENT
PROCESSING UNIT

1977 JUN 6 AM 10 30