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 KEISER, H. W. Pennsylvania Power & Light Co.  
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 BUTLER, W. R. Project Directorate I-2

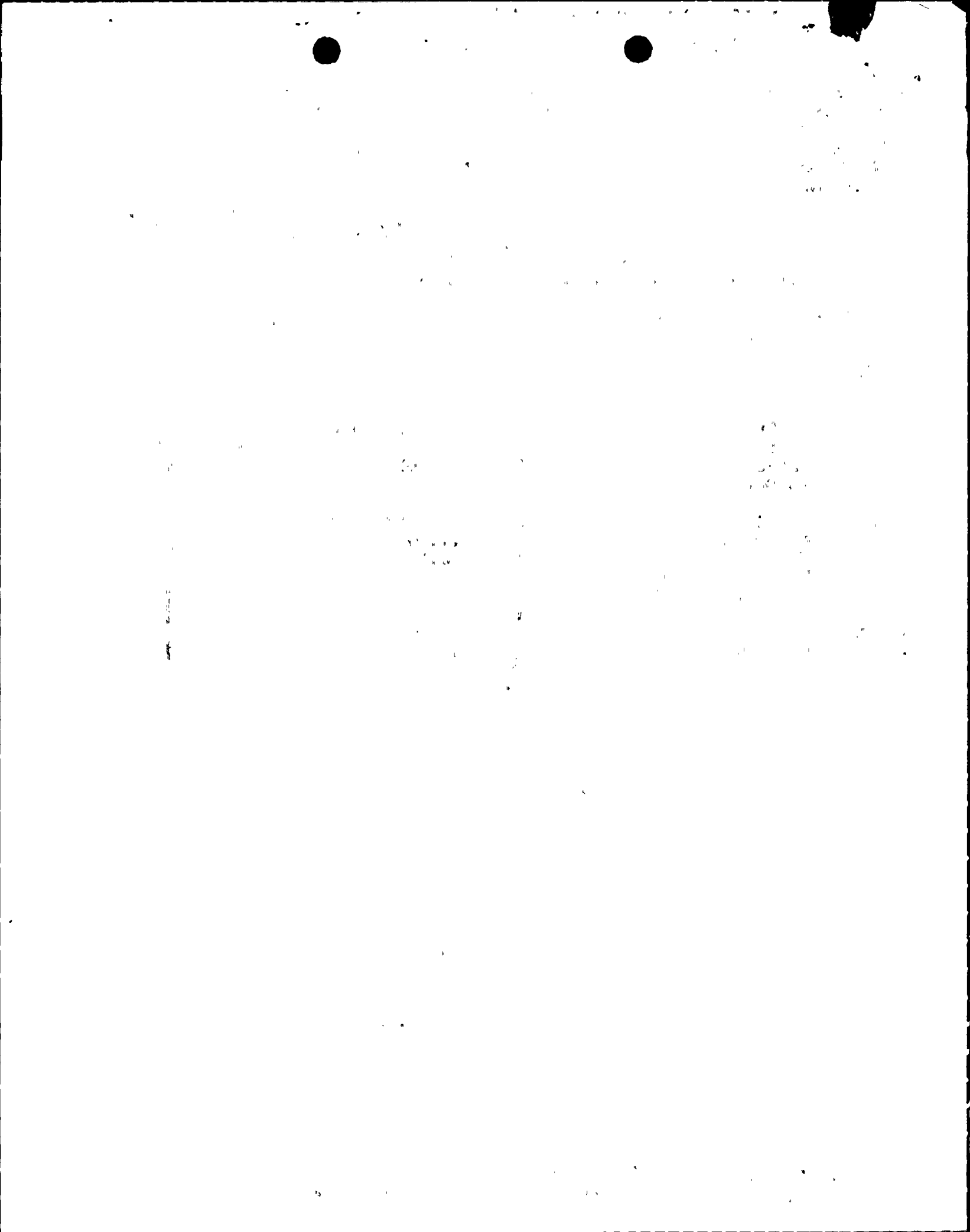
SUBJECT: Corrects documentation transmitted w/870619 application for amend to License NPF-14, revising Tech Specs to support Cycle 4 operations. Info includes Cycle 4 stability analysis & control rod drop accident analysis.

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Harold W. Keiser  
Vice President-Nuclear Operations  
215/770-7502

OCT 15 1987

Director of Nuclear Reactor Regulation  
Attention: Dr. W.R. Butler, Project Director  
Project Directorate I-2  
Division of Reactor Projects  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

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SUSQUEHANNA STEAM ELECTRIC STATION  
CORRECTIONS TO PROPOSED AMENDMENT 100  
TO LICENSE NO. NPF-14  
PLA-2930 FILES A7-8C, A17-2, R41-2

Docket No. 50-387

Reference: PLA-2875, letter from H.W. Keiser to W.R. Butler, dated  
June 19, 1987.

Dear Dr. Butler:

The referenced letter proposed changes to the Susquehanna SES Unit 1 Technical Specifications in support of Cycle 4 operation. The purpose of this letter is to provide corrections to some of the documentation that was transmitted with that proposal. The subject documentation did not form the basis for any of the proposed changes to the Technical Specifications; it was merely part of the analysis that supported Cycle 4 operation and therefore was provided for completeness in order to facilitate your review. Consequently, no changes to the proposed Technical Specification revisions in the referenced letter are being requested due to the corrections, which are described below. This information was transmitted to your staff via telecon on September 25, 1987.

Unit 1 Cycle 4 Stability Analysis

The Advanced Nuclear Fuels (ANF) calculated decay ratio for the Unit 1 Cycle 4 stability analysis of the 68/45 power/flow statepoint has changed from 0.66 to 0.70. This change originates from correcting an error in the Unit 1 Cycle 4 void coefficient for the 68/45 power/flow statepoint.

Unit 1 Cycle 4 Control Rod Drop Accident (CRDA) Analysis

The ANF calculated peak deposited enthalpy for the Unit 1 Cycle 4 CRDA analysis has changed from 91 to 191 cal/gm and the associated number of fuel rods exceeding 170 cal/gm changed from 0 to <60. This change in peak deposited enthalpy and number of fuel rods exceeding 170 cal/gm arises from the use of a more conservative control rod pattern in the CRDA analysis.

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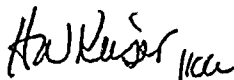
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FILES A7-8C, A17-2, R41-2 PLA-2930  
Dr. W.R. Butler

Since the FSAR (Section 15.4.9) assumes 770 rods fail due to a greater than 170 cal/gm fuel rod enthalpy, the radioactive releases for Unit 1 Cycle 4 are less than those calculated in the bounding FSAR analysis (Section 15.4.9).

Any questions on these corrections may be directed to Mr. R. Sgarro at (215) 770-7916.

Very truly yours,



H. W. Keiser  
Vice President-Nuclear Operations

Attachments

cc: NRC Document Control Desk (original)  
NRC Region I  
Mr. L. R. Plisco, NRC Resident Inspector  
Mr. M. C. Thadani, NRC Project Manager  
Mr. T. M. Gerusky, Pa DER

