REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 8707060658 DOC. DATE: 87/06/30 NOTARIZED: NO DOCKET # FACIL: 50-387 Susquehanna Steam Electric Station, Unit 1, Pennsylva 05000387

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RECIP. NAME RECIPIENT AFFILIATION

SUBJECT: LER 83-130/01T-1: on 831007, review of Tech Specs uncovered discrepancies between FSAR, Tech Specs & design drawings re containment isolation valves isolation signal. Caused by lack of communication. Appropriate changes made. W/870630 ltr.

DISTRIBUTION CODE: IE22D COPIES RECEIVED: LTR / ENCL / SIZE: TITLE: 50.73 Licensee Event Report (LER), Incident Rpt, etc.

NOTES: 1cy NMSS/FCAF/PM. LPDR 2cys Transcripts.

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	THADANI, M	1	1			
INTERNAL:	ACRS MICHELSON	1	1	ACRS MOELLER	2	2
	AEOD/DOA	1	1	AEOD/DSP/ROAB	2	2
	AEOD/DSP/TPAB	1	1	DEDRO	1	1
	NRR/DEST/ADE	1	0	NRR/DEST/ADS	1	0
	NRR/DEST/CEB	1	ļ	NRR/DEST/ELB	1	1
	NRR/DEST/ICSB	1	1	NRR/DEST/MEB	1	1
	NRR/DEST/MTB	1	1	NRR/DEST/PSB	1	1
	NRR/DEST/RSB	1	1	NRR/DEST/SGB	1	1
	NRR/DLPQ/HFB ,	1	1	NRR/DLPQ/QAB	1	1
	NRR/DDEA/EAB	1	1	NRR/DREP/RAB	1	1
	NRR/DREP/RPB	2	2	NRR/PMAS/ILRB	1	1
	NRR/PMAS/PTSB	1	1 <	REG FILE 02	1	1
	RES DEPY GI	1	1	RES TELFORD, J	1	1
	RES/DE/EIB	1	1	RGN1 FILE 01	1	1
EXTERNAL:	EG&G GROH, M	5	5	H ST LOBBY WARD	1	1
	LPDR	2	2	NRC PDR '	1	1
	NSIC HARRIS, J	1	1	NSIC MAYS, G	1	1
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NOTES:

NAC ((12-81 10 CF		U.S. NUCLEAR REGULATORY COMMISSION LICENSEE EVENT REPORT	APPROVED BY OMB 3150-0011
	CONTROL BLOCK:	1) IPLEASE PRINT OR TYPE ALL RE	QUIRED INFORMATION)
<u>[01</u>	PA SES 1000	- 000000 - 000 34 1	1 1 1 1 0
0 1	REPORT L @ 05 00		0 6 3 0 8 7 0
0 2	EVENT DESCRIPTION AND PROBABLE CO A review of Technical Sp	nsequences (10) ecifications uncovered discrepanci	es between
0 3	the FSAR, Technical Spec	ifications, and Design Drawings,pe	rtaining to contain-
0 4	ment isolation valves' i	solation signals. Twenty-one valv	es have been found
0 5	with discrepancies. Two	of these valves required a modifi	cation to the valves'
0 6	circuitry. These valves	are not used in normal plant oper	ation and were closed
0 7	and deactivated until the	e modifications were complete.	
0 8	ļ		
0 9	SD 11 X 12	CAUSE SUBCODE COMPONENT CODE SUBCODE Z 13 Z Z Z Z Z Z Z Z Z Z Z Z Z Z Z Z Z Z	SUBCODE (3) Z (8)
10	TERPORT 8 3 3 3 3 3 3 3 3 3	(3) 10 0 0 0 1 1 (3) 1 1 (3) 1 1 (4)	rime comp. component 26 Z 23 Z 9 9 9 9 43 revised FSAR,
	Technical Specifications	, and updated plant drawings cause	d a breach in the
1 2	communications channel be	etween the NSSS Vendor, the Archite	ect/Engineer, and
13	the Licensee. Appropriate	te changes were made to the FSAR, !	Tech. Specs.,
114	Plant Drawings and valve		
115	FACILITY	9	fications Review
,	8 9 10 12 13 ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIV	<u> </u>	PRELEASE 36
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210	ISSUED DESCRIPTION 49	5	NRC USE ONLY
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NAC Form	366A

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSIO	N
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APPROVED OMB NO. 3150-0104 EXPIRES: 8/31/88

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6) PAGE (3)
Susquehanna Steam Electric Station		YEAR SEQUENTIAL REVISION NUMBER
Unit 1	0 5 0 0 0 3 8	7 8 3 _ 1 3 0 _ 9 1 0 2 0 6 0 1

TEXT (If more space is required, use additional NRC Form 366A's) (17)

EVENT DESCRIPTION

At the close of the Construction Phase, just prior to Licensing, PP&L notified the Commission that discrepancies concerning the closure signals to various containment isolation valves existed among the plant's Technical Specifications (Tech Spec.), Final Safety Analysis Report (FSAR) and the "As Built" condition of the plant. The signals on which twenty-one installed isolation valves close; did not match the signals specified in the Plant's Technical Specifications and FSAR. The affected valves were closed (except for two valves which was found to isolate on a more conservative signal) pending permanent corrective action. There were no similar discrepancies found involving Unit Two valves.

The initial notification occurred on October 7, 1983. PP&L personnel informed the Commission that 10 isolation valves had been discovered whose isolation signals differed with those listed in the Technical Specifications and the FSAR. Licensee Event Report (LER) 83-130/01T-0, which further detailed the discrepancies, was filed on October 21, 1983. Between the time of the initial notification and the LER, similar discrepancies were found on eleven additional valves.

Table I, attached to this report, provides a list of the valves involved, the signals (Technical Specification, FSAR, and plant drawings) on which the valves would have closed, and the corrective action taken. As stated in the written follow-up report for the October 7, 1983 verbal notification, two valves were discovered to have no isolation signal. The affected valves were the RHR-Shutdown Cooling Return/LPCI Injection Testable Check Valve Bypass valves, HV-151-F122A&B. Amendment No. 23 to the Unit 1 Technical Specifications reclassified the valves as manual isolation valves. Manual isolation valves do not require automatic isolation signals.

A total of seven valves (five TIP ball isolation valves and two RHR-Rx vessel head spray valves) required circuit modifications to correct the discrepancies. The Traversing Incore Probe (TIP) isolation valves (5) required a modification to change their logic so that they isolated on Reactor Vessel level 3 and/or Hi Drywell pressure. The modification was completed October 19, 1983. Prior to the modification the valves closed on Reactor Vessel level 2 and/or Hi Drywell pressure. The two RHR-Reactor Vessel Head Spray Valves, HV-151-F022 & 23, would isolate on Reactor Vessel low Level 3; however, they would not isolate on Hi Drywell Pressure. A modification was completed May 4, 1984 which added the logic necessary to close the valves on Hi Drywell Pressure. Technical Specifications changes were made which brought the remaining 12 valves' isolation signals into conformance with the As Built condition of the plant, which was the approved design.

TABLE I

VALVES		SPEC TABLE 3-1 SIGNAL	TECH SPEC TABLE 3.3.2-1 SIGNAL	FSAR TABLE 18.1-12	DWGS	IMMEDIATE CORRECTIVE ACTION	PERMANENT CORRECTIVE ACTION
RHR-SHUTDOWN COOLING RETURN LPCI INJECTION PRESSURE EQUAL VALVES HV-151-F122A&	n/ N LIZING	Z .	Z-HI DW PRESS	IOW LEVEL 3 Z-HI DW Press	NONE		TECH SPEC CHANGED TO INDICATE F122A&B TO BE MANUAL ISOLATION VALVES (AMEND. NO. 23)
5 TRAVERSING INCORE PROBE BALL VALVES			Z-HI DW PRESS	LOW LEVEL 3 Z-HI DW PRES.	LOW LEVEL 2 HI DW PRESS	CLOSE VALVES	MODIFICATION TO LOGIC SO THAT VALVES CLOSE ON LEVEL 3 SIGNAL. TECH SPEC CHANGED TO REQUIRE VALVES TO CLOSE ON LOW LEVEL 3 SIGNAL OR HI DW PRESSURE (AMEND. NO. 36)
RHR-REACTOR V. HEAD SPRAY VA HV-151-F022&2	LVES	2	Z-HI DW PRESS	LOW LEVEL 3 Z-HI DW PRES	LOW LEVEL 3	CLOSE VALVES	MODIFICATION TO LOGIC SO THAT VALVES CLOSE ON HI DW PRESSURE SIGNAL, TECH SPEC CHANGED TO REQUIRE VALVES TO CLOSE ON LOW LEVEL 3 SIGNAL OR HI DW PRESSURE SIGNAL (AMEND. NO. 36)
RHR-SHUTDOWN COOLING SUCTI HV-151-F008&9		Z	Z-HI DW PRESS	IOW LEVEL 3 Z-HI DW PRES.	LOW LEVEL 3	CLOSE VALVES	TECH SPEC CHANGED TO REQUIRE VALVES TO CLOSE ON LOW LEVEL 3 SIGNAL (AMEND NO. 36)

			SPEC TABLE 3-1 SIGNAL	TECH SPEC TABLE 3.3.2-1 SIGNAL	FSAR TABLE 18.1-12	DWGS	IMMEDIATE CORRECTIVE ACTION	PERMANENT CORRECTIVE ACTION	
	REACTOR WATER CLEANUP SUCTION VALVES HV-144-F001 &		A .	A-LOW LEVEL 3	A-LOW LEVEL 3	LOW LEVEL 2	CLOSE VALVES	TECH SPEC CHANGED TO REQUIRE VALVES TO CLOSE ON LOW LEVEL 2 SIGNAL (AMEND NO. 18)	
•	RHR-SUPPRESSIC POOL COOLING/ SPRAY VALVES HV-151-F011A&E		B&C	B-LOW LEVEL 2 C-MAIN STEAM LINE HI RAD	B-LOW LEVEL 2 C-MAIN STEAM LINE HI RAD	LOW LEVEL 1 HI DW PRESS	CLOSE VALVES	TECH SPEC CHANGED TO REQUIRE VALVES TO CLOSE ON LOW LEVEL 1 OR HI DW PRESSURE SIGNAL (AMEND NO. 23)	-
	LIQUID RADWAST VALVES HV-16108 A1&A2 HV-16116 A1&A2	2	Z	Z-HI DW PRES	LOW LEVEL 3 HI DW PRES	LOW LEVEL 2 HI DW PRES	CLOSE VALVES	TECH SPEC CHANGED TO REQUIRE VALVES TO CLOSE ON LOW LEVEL 2 OR HI DW PRESS (AMEND. NO. 36)	
	SUPPRESSION PO CLEANUP VALVES HV-15766&68		У .	Y-HI DW PRES	LOW LEVEL 2 HI DW PRES	HI DW PRES	NONE (PRESENT LOGIC ORE CONSERVATIVE HAN TECH. SPEC.)	TECH SPEC CHANGED TO REQUIRE VALVES TO CLOSE ON LOW LEVEL 3 OR HI DW PRESS (AMEND NO. 36)	

June 30, 1987

U. S. Nuclear Regulatory Commission Document Control Desk Washington, DC 20555

SUSQUEHANNA STEAM ELECTRIC STATION LICENSEE EVENT REPORT 83-130/01T-1 FILE R41-2 PLAS - 263

Docket No. 50-387 License No. NPF-14

Attached is Licensee Event Report 83-130/01T-1 which is an update to the interim report filed October 21, 1983. This event was determined reportable per Technical Specification 6.9.1.8.i, in that during the course of reviewing of the Unit 2 Technical Specifications, discrepancies were identified in the use of characters for isolation signals to 21 containment isolation valves within the Unit 1 Technical Specifications.

R. G. Byram

Superintendent of Plant - Susquehanna

RGB/cmw

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IE37