



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PENNSYLVANIA POWER & LIGHT COMPANY

ALLEGHENY ELECTRIC COOPERATIVE, INC.

DOCKET NO. 50-387

SUSQUEHANNA STEAM ELECTRIC STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 65
License No. NPF-14

1. The Nuclear Regulatory Commission (the Commission or the NRC) having found that:
 - A. The application for the amendment filed by the Pennsylvania Power & Light Company, dated May 14, 1987, as supplemented by letters dated May 19 and 21, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-14 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 65 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. PP&L shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

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PDR ADOCK 05000387
P PDR



3. This license amendment became effective May 22, 1987.

FOR THE NUCLEAR REGULATORY COMMISSION

/s/

Bruce A. Boger, Assistant Director
for Region I Reactors
Division of Reactor Projects I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 28, 1987

PDI-2/A
NOV 1987
5/24/87

MO
PDI-2/PM
MThadani
5/26/87

OGC
Woodhead
5/26/87

WB
PDI-2/D
WButler
5/28/87

BB
AD/RI
BBoger
5/28/87

1982

1983

1984

1985

3. This license amendment became effective May 22, 1987.

FOR THE NUCLEAR REGULATORY COMMISSION



Bruce A. Boger, Assistant Director
for Region I Reactors
Division of Reactor Projects I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 28, 1987

ATTACHMENT TO LICENSE AMENDMENT NO. 65

FACILITY OPERATING LICENSE NO. NPF-14

DOCKET NO. 50-387

Replace the following pages of the Appendix A Technical Specifications with enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The overleaf page is provided to maintain document completeness.

REMOVE

3/4 6-19
3/4 6-20

INSERT

3/4 6-19
3/4 6-20 (overleaf)



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TABLE 3.6.3-1

PRIMARY CONTAINMENT ISOLATION VALVES

<u>VALVE FUNCTION AND NUMBER</u>	<u>MAXIMUM ISOLATION TIME (Seconds)</u>	<u>ISOLATION SIGNAL(s)^(a)</u>
<u>a. Automatic Isolation Valves</u>		
<u>MSIV</u>		
HV-141F022 A,B,C,D	5	X,C,D,E,P,UA
HV-141F028 A,B,C,D	5	X,C,D,E,P,UA
<u>MSL Drain</u>		
HV-141F016	10	X,C,D,E,P,UA
HV-141F019	10	X,C,D,E,P,UA
<u>RCIC Steam Supply</u>		
HV-149F007	20	K,KB
HV-149F008	20	K,KB
HV-149F088	3	K,KB
<u>HPCI Steam Supply</u>		
HV-155F002*	50	L, LB
HV-155F003	50	L, LB
HV-155F100	3	L, LB
<u>RHR - Shutdown Cooling Suction</u>		
HV-151F008	52	A,M,UB
HV-151F009	52	A,M,UB
<u>RWCU Suction^(b)</u>		
HV-144F001	30	B,J,W
HV-144F004	30	I,B,J,W
<u>RHR - Reactor Vessel Head Spray</u>		
HV-151F022	30	A,M,UB,Z
HV-151F023	20	A,M,UB,Z

*The HPCI HV-155F002 valve may be considered OPERABLE with its current minimum torque switch setting for the period beginning May 23, 1987 until an outage of sufficient duration to revise the setting occurs but no later than the next refueling outage scheduled on or about September 12, 1987.

TABLE 3.6.3-1 (Continued)

PRIMARY CONTAINMENT ISOLATION VALVES

<u>VALVE FUNCTION AND NUMBER</u>	<u>MAXIMUM ISOLATION TIME (Seconds)</u>	<u>ISOLATION SIGNAL(s)^(a)</u>
<u>Automatic Isolation Valves (Continued)</u>		
<u>Containment Instrument Gas</u>		
HV-12603	20	X,Z
SV-12605	N/A	X,Z
SV-12651	N/A	X,Z
SV-12661	N/A	Y,B
SV-12671	N/A	Y,B
<u>RBCCW</u>		
HV-11313	30	X,Z
HV-11314	30	X,Z
HV-11345	30	X,Z
HV-11346	30	X,Z
<u>Containment Purge</u>		
HV-15703	15	B,Y,R
HV-15704	15	B,Y,R
HV-15705	15	B,Y,R
HV-15711	15	B,Y,R
HV-15713	15	B,Y,R
HV-15714	15	B,Y,R
HV-15721	15	B,Y,R
HV-15722	15	B,Y,R
HV-15723	15	B,Y,R
HV-15724	15	B,Y,R
HV-15725	15	B,Y,R
<u>RHR - Drywell Spray^(c)</u>		
HV-151F016 A,B	90	X,Z
<u>RB Chilled Water</u>		
HV-18781 A1,A2,B1,B2	40	X,Z
HV-18782 A1,A2,B1,B2	6	X,Z
HV-18791 A1,A2,B1,B2	15	Y,B
HV-18792 A1,A2,B1,B2	4	Y,B