

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 8705180375 · DOC. DATE: 87/05/13 NOTARIZED: NO DOCKET #
 FACIL: 50-388 Susquehanna Steam Electric Station, Unit 2, Pennsylv 05000388
 AUTH. NAME AUTHOR AFFILIATION
 HIRT, J. A. Pennsylvania Power & Light Co.
 BYRAM, R. G. Pennsylvania Power & Light Co.
 RECIP. NAME RECIPIENT AFFILIATION

SUBJECT: LER 87-007-00: on 870413, Limiting Condition for Operation
 3.5.1 entered on HPCI sys to perform surveillance test.
 Caused by HPCI turbine sys supply valve trip due to stuck
 contacts on relay 2R. Relay replaced. W/870513 ltr.

DISTRIBUTION CODE: IE22D COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 4
 TITLE: 50.73 Licensee Event Report (LER), Incident Rpt, etc.

NOTES: 1cy NMSS/FCAF/PM. LPDR 2cys Transcripts. 05000388

	RECIPIENT ID CODE/NAME	COPIES LTTR ENCL	RECIPIENT ID CODE/NAME	COPIES LTTR ENCL
	PD1-2 LA THADANI, M	1 1 1 1	PD1-2 PD	1 1
INTERNAL:	ACRS MICHELSON	1 1	ACRS MOELLER	2 2
	AEOD/DOA	1 1	AEOD/DSP/ROAB	2 2
	AEOD/DSP/TPAB	1 1	DEDRO	1 1
	NRR/DEST/ADE	1 0	NRR/DEST/ADS	1 0
	NRR/DEST/CEB	1 1	NRR/DEST/ELB	1 1
	NRR/DEST/ICSB	1 1	NRR/DEST/MEB	1 1
	NRR/DEST/MTB	1 1	NRR/DEST/PSB	1 1
	NRR/DEST/RSB	1 1	NRR/DEST/SGB	1 1
	NRR/DLPQ/HFB	1 1	NRR/DLPQ/GAB	1 1
	NRR/DOEA/EAB	1 1	NRR/DREP/EPB	1 1
	NRR/DREP/RAB	1 1	NRR/DREP/RPB	2 2
	NRR/PMAS/ILRB	1 1	NRR/PMAS/PTSB	1 1
	<u>REG FILE</u> 02	1 1	RES DEPY GI	1 1
	RGNI FILE 01	1 1		
EXTERNAL:	EG&G GROH, M	5 5	H ST LOBBY WARD	1 1
	LPDR	2 2	NRC PDR	1 1
	NSIC HARRIS, J	1 1	NSIC MAYS, G	1 1
NOTES:		3 3		

TOTAL NUMBER OF COPIES REQUIRED: LTTR 47 ENCL 45

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Susquehanna Steam Electric Station - Unit Two	DOCKET NUMBER (2) 0 5 0 0 0 3 8 1	PAGE (3) / 1 OF 0 3
TITLE (4) The High Pressure Coolant Injection System Declared Inoperable Due To A Malfunctioning Steam Supply Valve		

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0	4	13	8	7	-	0	5	13			0 5 0 0 0
0	4	13	8	7	-	0	5	13			0 5 0 0 0

OPERATING MODE (9) 1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)					
POWER LEVEL (10) 1 0 0	<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.405(e)	<input type="checkbox"/> 60.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)	OTHER (Specify in Abstract Below and in Text, NRC Form 366A)	
	<input type="checkbox"/> 20.405(a)(1)(i)	<input type="checkbox"/> 60.38(e)(1)	<input type="checkbox"/> 60.73(a)(2)(v)	<input type="checkbox"/> 73.71(e)		
	<input type="checkbox"/> 20.405(a)(1)(ii)	<input type="checkbox"/> 60.38(e)(2)	<input checked="" type="checkbox"/> 60.73(a)(2)(vii)			
	<input type="checkbox"/> 20.405(a)(1)(iii)	<input type="checkbox"/> 60.73(a)(2)(i)	<input type="checkbox"/> 60.73(a)(2)(viii)(A)			
	<input type="checkbox"/> 20.405(a)(1)(iv)	<input type="checkbox"/> 60.73(a)(2)(ii)	<input type="checkbox"/> 60.73(a)(2)(viii)(B)			
	<input type="checkbox"/> 20.405(a)(1)(v)	<input type="checkbox"/> 60.73(a)(2)(iii)	<input type="checkbox"/> 60.73(a)(2)(x)			

LICENSEE CONTACT FOR THIS LER (12)		TELEPHONE NUMBER	
NAME Jeffrey A. Hirt, Engineer Level I		AREA CODE 7 1 7	NUMBER 5 4 2 - 3 9 1 7

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)										
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS
X	3 N	*	G 0 8 0	Y						

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input checked="" type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input type="checkbox"/> NO		0	6	2 6 8 7

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On April 13, 1987, at 1701 hours, Limiting Condition for Operation (LCO) 3.5.1 was entered on the High Pressure Coolant Injection (HPCI) system in order to perform surveillance test SO-252-004 'Quarterly HPCI Valve Exercising.' At the time the unit was operating at 100% rated thermal power. At approximately 1800 hours Operations personnel attempted to cycle the HPCI Turbine Steam Supply Valve, HV-255-F001. The valve opened satisfactorily. When Operations personnel tried to close HV-255-F001 the valve's supply breaker tripped. As a result, the HPCI system was declared inoperable for reasons other than surveillance testing.

Electrical Maintenance personnel investigated and found that contacts on the 2R relay were stuck in the closed position. The 2R relay energizes, closing its contacts, to open the valve. When the operator attempted to close the valve the abnormal circuit configuration, caused by the contacts being closed, resulted in the tripping of the supply breaker.

Electrical maintenance personnel freed the 2R relay contacts and stroked the supply valve several times. The valve operated satisfactorily. Based on this, Operations personnel cleared LCO 3.5.1 at 2300 hours on April 13, 1987. Surveillance test SO-252-004 was successfully completed on April 20, 1987 at 1800 hours. On April 21, 1987 Electrical Maintenance personnel replaced the contacts on the 2R relay. Investigation is continuing concerning this event. The results will be presented in an update.

* Not Listed

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Susquehanna Steam Electric Station Unit Two	DOCKET NUMBER (2) 0 5 0 0 0 3 8 8 8 7	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		- 0	0 7	- 0 0	0 2	OF 0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

DESCRIPTION

On April 13, 1987, at 1701 hours, Limiting Condition for Operation (LCO) 3.5.1 was entered on the High Pressure Coolant Injection (HPCI) system (EIIS Code: BJ) in order to perform surveillance test SO-252-004 'Quarterly HPCI Valve Exercising.' At the time the unit was operating at 100% rated thermal power. At approximately 1800 hours Operations personnel attempted to cycle the HPCI Turbine Steam Supply Valve, HV-255-F001. The valve opened satisfactorily. When Operations personnel tried to close HV-255-F001 the valve's supply breaker tripped. As a result, the HPCI system was declared inoperable for reasons other than surveillance testing.

Electrical Maintenance personnel investigated and found that contacts on the 2R relay were stuck in the closed position. The 2R relay energizes, closing its contacts, to open the valve. When the operator attempted to close the valve the abnormal circuit configuration, caused by the contacts being closed, resulted in the tripping of the supply breaker.

CAUSE/CORRECTIVE ACTION

The supply breaker to the HPCI turbine steam supply valve tripped due to contacts on the 2R relay being stuck together while the operator attempted to shut the valve. Electrical Maintenance personnel freed the contacts and stroked the supply valve several times. The valve operated satisfactorily. Based on this, Operations personnel cleared LCO 3.5.1 at 2300 hours on April 13, 1987. On April 20, 1987 at 1800 hours surveillance test SO-252-004 was successfully completed. On April 21, 1987 the contacts on the 2R relay were replaced. Investigation is continuing concerning this event. The results will be presented in an update.

SAFETY SIGNIFICANCE

This event did not pose a significant safety consequence. The HPCI system was out of service for approximately six hours. The Action Statement to LCO 3.5.1 allows the HPCI system to be inoperable for up to 14 days provided the Core Spray System (CSS) (EIIS Code: BM), the Low Pressure Coolant Injection (LPCI) system (EIIS Code: BO), the Automatic Depressurization System (ADS) (EIIS Code: Not listed) and the Reactor Core Isolation Cooling (RCIC) system (EIIS Code: BN) are operable. The CSS, LPCI system, ADS and the RCIC system were operable during this six hour period.

No other plant status was identified in which this safety consequence would have been more severe.

SIMILAR OCCURRENCES

There were no past Licensee Event Reports identified which reported to the Commission an instance where the HPCI system became operable due to the contacts of the 2R relay being stuck together.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Susquehanna Steam Electric Station Unit Two	DOCKET NUMBER (2) 0 5 0 0 0 3 8 8 8 7	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
			- 0 0 7	- 0 0	0 3	OF	0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

REPORTABILITY

This event was determined to be reportable to the Commission per 10CFR50.73 (a) (2) (vii), in that a single train safety system, HPCI, became inoperable due to a failure within the control circuitry of the steam supply valve.



Pennsylvania Power & Light Company

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May 13, 1987

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

SUSQUEHANNA STEAM ELECTRIC STATION
LICENSEE EVENT REPORT 87-007-00
FILE R41-2
PLAS- 252

Docket No. 50-388
License No. NPF-22

Attached is Licensee Event Report 87-007-00. This event was determined reportable per 10CFR50.73 (a) (2) (vii), in that the High Pressure Coolant Injection System, a single train safety system, was inoperable for six hours due to an electrical failure.


R.G. Byham
Superintendent of Plant-Susquehanna

JAH/cdn

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