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AUTH NAME	AUTHOR AFFILIATION	
CURTIS, N.W.	Pennsylvania Power & Light Co.	
RECIP, NAME	RECIPIENT AFFILIATION	
SCHWENCER, A,	Licensing Branch 2	

SUBJECT: Forwards relief requests from preservice insp program, consisting of Rev 4 to Relief Request 4 & Rev 2 to Relief Request 6 adding four welds in unexamined code vol.

NOTES:1cy NMSS/FCAF/PM. LPDR 2cys Transcripts. OL:07/17/82

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Pennsylvania Power & Light Company

Two North Ninth Street • Allentown, PA 18101 • 215 / 770-5151

Norman W. Curtis Vice President-Engineering & Construction-Nuclear 215/770-7501

DEC 1 9 1984

Director of Nuclear Reactor Regulation Attention: Mr. A. Schwencer, Chief Licensing Branch No. 2 Division of Licensing U.S. Nuclear Regulatory Commission Washington, D.C. 20555

SUSQUEHANNA STEAM ELECTRIC STATION Unit 1 PSI Program Relief Requests ER 100450 FILE 841-4 & 899 PLA-2374

Docket No. 50-387

References: (1) PLA-2004 dated 12/21/83 (2) PLA-1551 dated 3/3/83

Dear Mr. Schwencer:

Attached is Revision 4 of Relief Request #4 and Revision 2 of Relief Request #6. The revision of Relief Request #4 covers the addition of four (4) welds requiring relief for small areas of unexamined code volume discovered during the review of the final report.

The revision of Relief Request #6 was identified after a discrepancy was noted between the Unit 1 and Unit 2 pump data relative to examination coverage. A supplemental surface examination was not performed as an alternate provision; however, in-service inspection requirements currently apply and selected welds will be inspected during the inspection interval as an indication of continued integrity. In addition, the welds will be subject to system pressure testing each inspection period (three periods make up the 10 year interval).

If you have any questions, please call.

Very truly yours,

N. W. Curtis Vice President-Engineering & Construction-Nuclear

Attachment

8412280163 841 PDR ADDCK 0500

cc:	М.	J.	Campagnone	N
	R.	H.	Jacobs	NI
	R.	A.	McBrearty	NI

RC - Washington RC - Susq. SES RC - Region I

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RELIEF REQUEST #4

			RI	CLIEF REQUEST #4	•		
WELD IDENTIFICATION NUMBER	CODE CATEGORY AND ITEM NUMBER	System	CONFIGURATION	NATURE OF OBSTRUCTION	% OF SCAN OBSTRUCTED	ASME SECTION III EXAMINATION	SAFETY IMPACT
VRR-B31-2-2R	BJ B4.5	Reactor Recirc.	Pipe to Tee	Instrumentatio Nozzle	on 2%	RT,PT	Reactor Coolant Pressure boundary leak detection system detects leakage. Plant technical specifications require plant shutdown with greater than 5 gpm.
VNB-B-21-FW-B4	BJ B4.5	Main Steam	Elbow to Valve	I-Beam	5%	RT,PT	Weld cannot be isolated From the reactor coolant pressure boundary; however leak detection systems detect leakage. Plant technical specifications require shutdown with leakage greater than 5 gpm.
VNB-21-1-FW-C4	BJ B4.5	Main Steam	Elbow to Valve	I-Beam	5%	RT,PT	Weld cannot be isolated From the reactor coolant pressure boundary; however leak detection systems detect leakage. Plant technical specifications require shutdown with leakage greater than 5 gpm.
DBB-129-1-FW-11 mt/j245c:ncr	CG C2.1	Main Steam	Cap. to Pipe	Branch Connection	7%	RT	Radiation monitor will Detect significant leakage. Leak can be isolated and plant shut- down accomplished through the use of safe shutdown systems or ECCS depending on size of leak.

REVISION 4 10/29/84

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PRESERVICE INSPECTION SUSQUEHANNA SES UNIT #1 RELIEF REQUEST #6

I. IDENTIFICATION OF COMPONENTS:

Class 1, Category BJ, pressure retaining welds in piping

Class 2, Category CF and CG pressure retaining welds in piping and pumps

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2.

II. CODE REQUIREMENT:

Category BJ - Table IWB-2600, Item Numbers B4.5, B4.6, B4.7 - of the ASME Code, 1974 Edition to Summer 1975 Addenda requires volumetric examination of 100%* of circumferential welds, longitudinal welds, and branch connections be performed completely as a preservice examination requirement prior to initial plant start-up.

Category CF/CG - Table IWC-2600, Item Numbers C2.1, C2.2, C2.3, C3.1 of the ASME Code, 1974 Edition to Summer 1975 Addenda requires volumetric examination of 100% and 50%*, respectively of circumferential discontinuity welds, longitudinal welds, and branch connection welds be performed completely as a preservice examination requirement prior to initial plant start-up.

*Excluding those exempt per IWB-1220, IWC-1220.

ASME Appendix III, Winter 1975 Addenda, requires an angle beam examination of the weld and required volume (the lesser of $\frac{1}{2}$ " or 1") be performed scanning both normal and parallel to the weld.

III. BASIS FOR RELIEF:

Relief is required from the ASME Section XI examination requirements on the basis of inaccessibility of the weld and required volume due to geometric configuration.

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WELD IDENTIFICATION NUMBER	CODE CATEGORY AND ITEM NUMBER	System -	CONFIGURATION	NATURE OF OBSTRUCTION	% OF SCAN OBSTRUCTED	ASME SECTION III EXAMINATION	ŚAFETY IMPACT	8.
1P202A,B,C,D		Residual Heat Removal (Pump Welds)	-				•	
-361-5-13	CF C3.1	11	Shell to Suction Nozzle	Limited Scan Due to Part Geometry	10%	RT	During normal plant power operation, the pump welds are not pressurized. During normal system operation.	,
-361-3-13	CF C3.1	11	Suction Nozzle to Flange	11	30%	RT .	welds are under a maximum pressure of 460 psig. Leak detection system detects	
-361-5-8	CF C3.1	11	Top Closure Plate to Shell	11	5%	RT	significant leakage; leakage can affect one (1) RHR loop. Plant can be	
-361-4-6	CF C3.1	11	Discharge Elbow to Flange	11	30%	RT	safely cooled down by unaffected RHR loop.	-
-361-1-5	CF C3.1	11	Shell to Head Hub Flange	11	45%	RT	11	
1P206,A,B,C,D		Core Spray (Pump Welds)				-	- -	
-361-5-13	CG C3.1	11	Shell to Suction Nozzle	Limited Scan Due to Part Geometry	10%	RT	During normal plant power operation, welds are not pressurized. During normal system operation, weld is	
-361-4-6	- CG C3.1		Discharge Elbow to Flange	11	20%	RT	under a maximum pressure of 475 psig. Leak detection system detects significant	
-361-5-6	CG C3.1		Discharge Elbow to Shell	11	10%	RT	leakage which can affect one (1) core spray loop. Plant can be safely cooled down by unaffected core spray loop.	

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