

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

MAY

ACCESSION NBR: 8312270088 DOC. DATE: 83/12/21 NOTARIZED: NO DOCKET #
 FACIL: 50-387 Susquehanna Steam Electric Station, Unit 1, Pennsylv 05000387
 50-388 Susquehanna Steam Electric Station, Unit 2, Pennsylv 05000388
 AUTH. NAME: CURTIS, N.W. AUTHOR AFFILIATION: Pennsylvania Power & Light Co.
 RECIP. NAME: SCHWENCER, A. RECIPIENT AFFILIATION: Licensing Branch 2

SUBJECT: Forwards revised Relief Requests 8, 9 & 10 re RCIC, RHR & feedwater sys for review & approval. Response to SER Item 0 ill complete.

DISTRIBUTION CODE: B001S COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 8
 TITLE: Licensing Submittal: PSAR/FSAR Amdts & Related Correspondence

NOTES: 1cy NMSS/FCAF/PM. LPDR 2cys. 05000387
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	RECIPIENT ID CODE/NAME	COPIES LTR ENCL	RECIPIENT ID CODE/NAME	COPIES LTR ENCL
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	NRR LB2 LA	1 0	PERCH, R. 01	1 1
INTERNAL:	ELD/HDS4	1 0	IE FILE	1 1
	IE/DEPER/EPB 36	3 3	IE/DEPER/IRB 35	1 1
	IE/DEQA/QAB 21	1 1	NRR/DE/AEAB	1 0
	NRR/DE/CEB 11	1 1	NRR/DE/EHEB	1 1
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	NRR/DHFS/HFEB40	1 1	NRR/DHFS/LQB 32	1 1
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	RGN1	3 3	RM/DDAMI/MIB	1 0
EXTERNAL:	ACRS 41	6 6	BNL (AMDTS ONLY)	1 1
	DMB/DSS (AMDTS)	1 1	FEMA-REP DIV 39	1 1
	LPDR 03	2 2	NRC PDR 02	1 1
	NSIC 05	1 1	NTIS	1 1
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THE
 UNITED STATES OF AMERICA
 DEPARTMENT OF THE ARMY
 OFFICE OF THE CHIEF OF STAFF
 WASHINGTON, D. C. 20315

MEMORANDUM FOR THE RECORD
 SUBJECT: [Illegible]

NO.	DATE	DESCRIPTION	INITIALS	STATUS
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Pennsylvania Power & Light Company

Two North Ninth Street • Allentown, PA 18101 • 215 / 770-5151

Norman W. Curtis
Vice President-Engineering & Construction-Nuclear
215/770-7501

DEC 21 1983

Director of Nuclear Reactor Regulation
Attention: Mr. A. Schwencer, Chief
Licensing Branch No. 2
Division of Licensing
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

SUSQUEHANNA STEAM ELECTRIC STATION
SER ITEM NO. 111
ERs 100450/100508 FILES 899/841-2
PLA-2004

Docket Nos. 50-387
50-388

References: (1) PLA-1770 dated 8/02/83
(2) PLA-1934 dated 11/01/83

Dear Mr. Schwencer:

Enclosed for your review and approval is an update of information submitted previously under References (1) and (2). Relief request numbers 8, 9, and 10 submitted under Reference (2) have been revised and these revisions are attached. This letter completes PP&L's response to SER Item No. 111.

Very truly yours,

N. W. Curtis
Vice President-Engineering & Construction-Nuclear

cc: R. L. Perch - NRC
L. R. Plisko - NRC

Attachment: Relief Request #8, pg. 2
Relief Request #9, pg. 6, 11, 12
Relief Request #10, pg. 6, 15, 16

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RELIEF REQUEST #8

<u>WELD IDENTIFICATION NO.</u>	<u>CODE CATEGORY & ITEM NO.</u>	<u>SYSTEM</u>	<u>CONFIGURATION</u>	<u>NATURE OF OBSTRUCTION</u>	<u>% OF SCAN OBSTRUCTED</u>	<u>ASME SECTION III EXAMINATION</u>	<u>SAFETY IMPACT</u>
DBB-221-1-FW1	CF C2.1	Reactor Core Isolation Cooling	Pipe to Valve	Wrapper Plate	100%	RT	Leak detection systems detect weld leakage, resulting in either manual or automatic isolation of leak. These lines are not required for normal safe shutdowns and alternate shutdown methods are available (e.g., feedwater, RCIC, main condenser).

1

RELIEF REQUEST #9 (Continued)

WELD IDENTIFICATION NO.	CODE CATEGORY & ITEM NO.	SYSTEM	CONFIGURATION	NATURE OF OBSTRUCTION	% OF SCAN OBSTRUCTED	ASME SECTION III EXAMINATION	SAFETY IMPACT
VRR-B31-3-10-M ¹	BJ B4.5	Recirc.	Longitudinal Seam	Branch Line	5%	RT/PT	Affects RCPB; weld cannot be isolated. However RCPB leak detection systems detect leakage. Plant Tech. Specs. require plant shutdown with leakage greater than 5 GPM.
VRR-B31-3-10-L ¹	BJ B4.5	Recirc.	Longitudinal Seam	Branch Line	5%	RT/PT	"
VRR-B31-4-10-P ¹	BJ B4.5	Recirc.	Longitudinal Seam	Branch Line	10%	RT/PT	"
VRR-B31-4-10-Q ¹	BJ B4.5	Recirc.	Longitudinal Seam	Branch Line	10%	RT/PT	"
DBA-201-2-FW34	BJ B4.5	Reactor Water Cleanup	Pipe to Elbow	Rigid Restraint	35%	RT/PT	Leak detection systems detect weld leakage, resulting in either manual or automatic isolation of leak. These lines are not required for normal safe shutdowns and alternate shutdown methods are available (e.g., feedwater, RCIC, main condenser).
VRR-B31-4-2W ¹	BJ B4.5	Recirc.	Longitudinal Seam	Hanger	5%	RT/PT	Affects RCPB; weld cannot be isolated. However, RCPB leak detection systems detect leakage. Plant Tech. Specs. require plant shutdown with leakage greater than 5 GPM.
VRR-B31-3-2W ¹	BJ B4.5	Recirc.	Longitudinal Seam	Lugs	2%	RT/PT	Affects RCPB; weld cannot be isolated. However, RCPB leak detection systems detect leakage. Plant Tech. Specs. require plant shutdown with leakage greater than 5 GPM.

1. Obstructed areas are located outside of the required examination area (12 inches from the intersection with the edge of a circumferential weld) for subsequent inservice inspections.

RELIEF REQUEST #9 (Continued)

WELD IDENTIFICATION NO.	CODE CATEGORY & ITEM NO.	SYSTEM	CONFIGURATION	NATURE OF OBSTRUCTION	% OF SCAN OBSTRUCTED	ASME SECTION III EXAMINATION	SAFETY IMPACT
DBB-221-3-FW2	CF C2.1	Reactor Core Isolation Cooling	Tee to Valve	1) Branch Line 2) Geometry	10%	RT	Leak detection systems detect weld leakage, resulting in either manual or automatic isolation of leak. These lines are not required for normal safe shutdowns and alternate shutdown methods are available (e.g., feedwater, RCIC, main condenser).
DBB-217-1-FW2	CG C2.1	High Pressure Coolant Injection	Pipe to Valve	Pipe Support	8%	RT	Leak detection systems detect weld leakage, resulting in either manual or automatic isolation of leak. These lines are not required for normal safe shutdowns and alternate shutdown methods are available (e.g., feedwater, RCIC, main condenser).
GBB-205-2-1A	CF C2.1	Residual Heat Removal	Reducer to Reducer	Welded Hanger	25%	RT	During normal plant power operation, welds are not under pressure. During normal system operation, significant leakage of weld can affect pressure boundary of one RHR loop. However, significant leakage is detectable by leak detection systems or loss of system function. Plant can be safely cooled down by unaffected RHR loop or main condenser.
DLA-204-1-FW1	BJ B4.5	Feedwater	Pipe to Valve	Welded Hanger	50%	RT/PT	Affects RCPB; weld cannot be isolated. However, RCPB leak detection systems detect leakage. Plant Tech. Specs. require plant shutdown with leakage greater than 5 GPM.
VNB-B21-3-20W	BJ B4.5	Main Steam	Pipe to Elbow	Lamination	2%	RT/PT	Affects RCPB; weld cannot be isolated. However, RCPB leak detection systems detect leakage. Plant Tech. Specs. require plant shutdown with leakage greater than 5 GPM.

RELIEF REQUEST #9 (Continued)

WELD IDENTIFICATION NO.	CODE CATEGORY & ITEM NO.	SYSTEM	CONFIGURATION	NATURE OF OBSTRUCTION	% OF SCAN OBSTRUCTED	ASME SECTION III EXAMINATION	SAFETY IMPACT
VBB-202-1-5A	CG C2.1	Control Rod Drive	Pipe to Elbow	Lamination	5%	RT	During normal operation of the CRD system, leakage will be detected by area radiation monitors. Leakage can be manually isolated.
DCA-210-2-FW2	BJ B4.5	Residual Heat Removal	Valve to Flued Head	Joint Configuration	30%	RT/PT	Affects RCPB; weld cannot be isolated. However, RCPB leak detection systems detect leakage. Plant Tech. Specs. require shutdown with leakage greater than 5 GPM.

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RELIEF REQUEST #10 (Continued)

WELD IDENTIFICATION NO.	CODE CATEGORY & ITEM NO.	SYSTEM	CONFIGURATION	NATURE OF OBSTRUCTION	% OF SCAN OBSTRUCTED	ASME SECTION III EXAMINATION	SAFETY IMPACT
DBA-214-1-FWA14	NA	Main Steam (Augmented)	Elbow to Branch	Part Geometry	30%	RT/PT	Affects reactor coolant pressure boundary (RCPB); however, inside containment isolation valve performs RCPB isolation function. Any significant RCPB leakage would be detected by leak detection systems, which alert plant personnel to inspect and shutdown plant using unaffected systems.
DBA-214-1-FW22	NA	Main Steam (Augmented)	Elbow to Tee	Part Geometry	30%	RT/PT	Affects reactor coolant pressure boundary (RCPB); however, inside containment isolation valve performs RCPB isolation function. Any significant RCPB leakage would be detected by leak detection systems, which alert plant personnel to inspect and shutdown plant using unaffected systems.
DBA-212-1-FW4	BJ B4.5	Main Steam	Tee to Tee	Part Geometry	30%	RT/PT	Affects RCPB; weld cannot be isolated. However RCPB leak detection systems detect leakage. Plant Tech. Specs. require plant shutdown with leakage greater than 5 GPM.
DBB-215-1-FW10	CF C2.1	Residual Heat Removal	Elbow to Valve	Part Geometry	10%	RT/PT	Leak detection systems detect weld leakage, resulting in either manual or automatic isolation of leak. These lines are not required for normal safe shutdowns and alternate shutdown methods are available (e.g., feedwater, RCIC, main condenser).

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RELIEF REQUEST #10 (Continued)

WELD IDENTIFICATION NO.	CODE CATEGORY & ITEM NO.	SYSTEM	CONFIGURATION	NATURE OF OBSTRUCTION	% OF SCAN OBSTRUCTED	ASME SECTION III EXAMINATION	SAFETY IMPACT
2P-206-A 361-5-13 B C D	CG C2.1	Core Spray (Pump)	Inlet Nozzle to Vertical Support Shell	Part Geometry	10%	RT	During normal plant power operation, weld is not pressurized. During normal system operation, weld is under a maximum pressure of 475 psig. Leak detection system detects significant leakage. Significant leakage of weld can affect one core spray loop. Plant can be safely cooled down by unaffected core spray loop.
2P-206-A 361-5-6 B C D	CG C2.1	Core Spray (Pump)	Elbow to Vertical Support Shell	Part Geometry	10%	RT	During normal plant power operation, weld is not pressurized. During normal system operation, weld is under a maximum pressure of 475 psig. Leak detection system detects significant leakage. Significant leakage of weld can affect one core spray loop. Plant can be safely cooled down by unaffected core spray loop.
GBB-201-1-FW2	CG C2.1	Core Spray	Valve to Pipe	Part Geometry	25%	RT	During normal plant power operation, weld is not pressurized. During normal system operation, weld is under a maximum pressure of 475 psig. Leak detection system detects significant leakage. Significant leakage of weld can affect one core spray loop. Plant can be safely cooled down by unaffected core spray loop.

RELIEF REQUEST #10 (Continued)

WELD IDENTIFICATION NO.	CODE CATEGORY & ITEM NO.	SYSTEM	CONFIGURATION	NATURE OF OBSTRUCTION	% OF SCAN OBSTRUCTED	ASME SECTION III EXAMINATION	SAFETY IMPACT
GBB-204-1-FW7	CF C2.1	Residual Heat Removal	Valve to Pipe	Part Geometry	5%	RT	During normal plant power operation, welds are not under pressure. During normal system operation, significant leakage of weld can affect pressure boundary of one RHR loop. However, significant leakage is detectable by leak detection systems or loss of system function. Plant can be safely cooled down by unaffected RHR loop or main condenser.
DBB-215-1-FW7	CF C2.1	Residual Heat Removal	Valve to Pipe	Part Geometry	5%	RT	Leak detection systems detect weld leakage, resulting in either manual or automatic isolation of the leak. These lines are not required for normal safe shutdowns and alternate shutdown methods are available.
DLA-202-1-FW6	BJ B4.5	Feedwater	Pipe to Tee	Part Geometry	15%	RT/PT	Affects RCPB; weld cannot be isolated. However, RCPB leak detection systems detect leakage. Plant Tech. Specs. require plant shutdown with leakage greater than 5 GPM.

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