- MEMORANDUM FOR: C. J. Heltemes, Director Office for Analysis and Evaluation of Operational Data

FROM: H. R. Denton, Director Office of Nuclear Reactor Regulation

SUBJECT: AEOD ENGINEERING EVALUATION REPORT ON EVENTS INVOLVING RESIDUAL HEAT REMOVAL SYSTEM AT SUSQUEHANNA UNIT 1

Reference: Engineering Evaluation Report No. AEOD/E315 JULY 7, 1987

The subject report (reference 1) addresses two recent events involving operation of the residual heat removal system in the shutdown cooling mode at Susquehanna Unit 1.

In accordance with our mutual understanding relative to the appropriate disposition of AEOD Engineering Evaluation Reports, we have distributed the subject reports to the appropriate NRR branches and the Project Manager of the plant in question for their review. They will review this information and make use of it in future evaluations and SRP revisions as appropriate.

Original Signed by L.R. Denton

Harold R. Denton, Director Office of Nuclear Reactor Regulation

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cc: K. Seyfrit A. Schwencer

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*PREVIOUS CONCURRENCE SEE NEXT PAGE

ORAB:DL* JTBeard 09/26/83	C:ORAB:DL* GHolahan 09/26/83	AD/SA:DL* FMiraglia 09/27/83	D:DL* DEisenhut 09/28/83	DD/NRR ECase 09/20/83	D:NRB HDenton DØ/3/83	
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MEMORANDUM FOR:

C. J. Heltemes, Director Office for Analysis and Evaluation of Operational Data

FROM:

H. R. Denton, Director Office of Nuclear Reactor Regulation

SUBJECT

AEOD ENGINEERING EVALUATION REPORT ON EVENTS INVOLVING RESIDUAL HEAT REMOVAL SYSTEM AT SUSQUEHANNA UNIT 1

Reference:

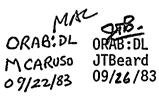
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> Harold R. Denton, Director Office of Nuclear Reactor Regulation

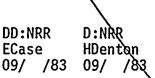
cc: K. Seyfrit A. Schwencher















UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

JUL 8 1983

MEMORANDUM FOR: Harold R. Denton, Director Office of Nuclear Reactor Regulation

FROM:

C. J. Heltemes, Jr., Director Office for Analysis and Evaluation of Operational Data

SUBJECT:

MISUSE OF VALVE RESULTING IN VIBRATION AND DAMAGE TO THE VALVE ASSEMBLY AND PIPE SUPPORTS

A recently completed Engineering Evaluation Report on the above subject is enclosed. Our evaluation concluded that severe damage to the LPCI system injection valve was directly related to RHR system flow limitations that result from a combination of system design, configuration, flow control system, and a low level of decay heat. Further, current intermittent operation of the RHR system in the shutdown cooling.mode has the potential for cumulative damage and may be related to the valve operator motor burnout that occurred in one of the events. We believe it would be appropriate to review RHR system operation for compatibility with valve assembly design and qualification requirements.

By separate memorandum, we are recommending that IE prepare an information notice to cover the extent of valve and pipe support damage and the system conditions that led to the misuse of the valve.

> C. H. Heltemest Jr., Director Office for Analysis and Evaluation of Operational Data

cc: R. C. DeYoung, IE E. L. Jordan, IE T. E. Murley, Region I

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