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 SCHWENCER, A. Licensing Branch 2

SUBJECT: Forwards Errata & Addenda Sheet 2 to NEDO-22128, "GE BWR
 Extended Load Line Limit Analysis for Susquehanna Steam
 Electric Station Unit 1," in response to NRC questions re
 discrepancies between FSAR & rept.

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Pennsylvania Power & Light Company

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Norman W. Curtis
Vice President-Engineering & Construction-Nuclear
215/770-7501

August 15, 1983

Director of Nuclear Reactor Regulation
Attention: Mr. A. Schwencer, Chief
Licensing Branch No. 2
Division of Licensing
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

SUSQUEHANNA STEAM ELECTRIC STATION
ADDITIONAL INFORMATION ON PROPOSED AMENDMENT 19
ER 100450 FILE 841-8
PLA-1792

Dear Mr. Schwencer:

The following discussion is in response to questions raised by Mr. S. Sun of the NRC Staff concerning a discrepancy between the Susquehanna FSAR and NEDO-22128 (Extended Load Line Limit Analysis for Susquehanna SES Unit 1).

The stability analysis performed for the Susquehanna FSAR in 1976 used a fuel gap conductance of 533.5 BTU/hr-ft²-°F. This is approximately the same value which is currently used correctly for the FSAR transient analyses. This gap conductance was based on the core average GEGAP analysis value, which was the procedure in use at that time.

Since that time, however, it was determined that a more appropriate value to use for the gap conductance is 1000 BTU/hr-ft²-°F, which is used in NEDO-22128. Justification for this value is described in General Electric NEDE-20944-P (BWR/4 and BWR/5 Fuel Design, 1976) and also in General Electric NEDE-23786-1-P (Fuel Rod Prepressurization Amendment 1, 1978) for the fuel design in use at Susquehanna SES. Both of these GE Company - Proprietary documents have been previously submitted to NRC.

Based on this information, FSAR Subsection 4.4.4.6.6 and Figure 4.4-6 will be changed (see revised Figure 4.4-6 attached). General Electric has also revised NEDO-22128 to be more consistent with the FSAR (also attached, 3 pages).

If you have any further questions on this matter, please contact us.

Very truly yours,

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P PDR

N. W. Curtis
Vice President-Engineering and Construction-Nuclear

Good
Boo! 1/1

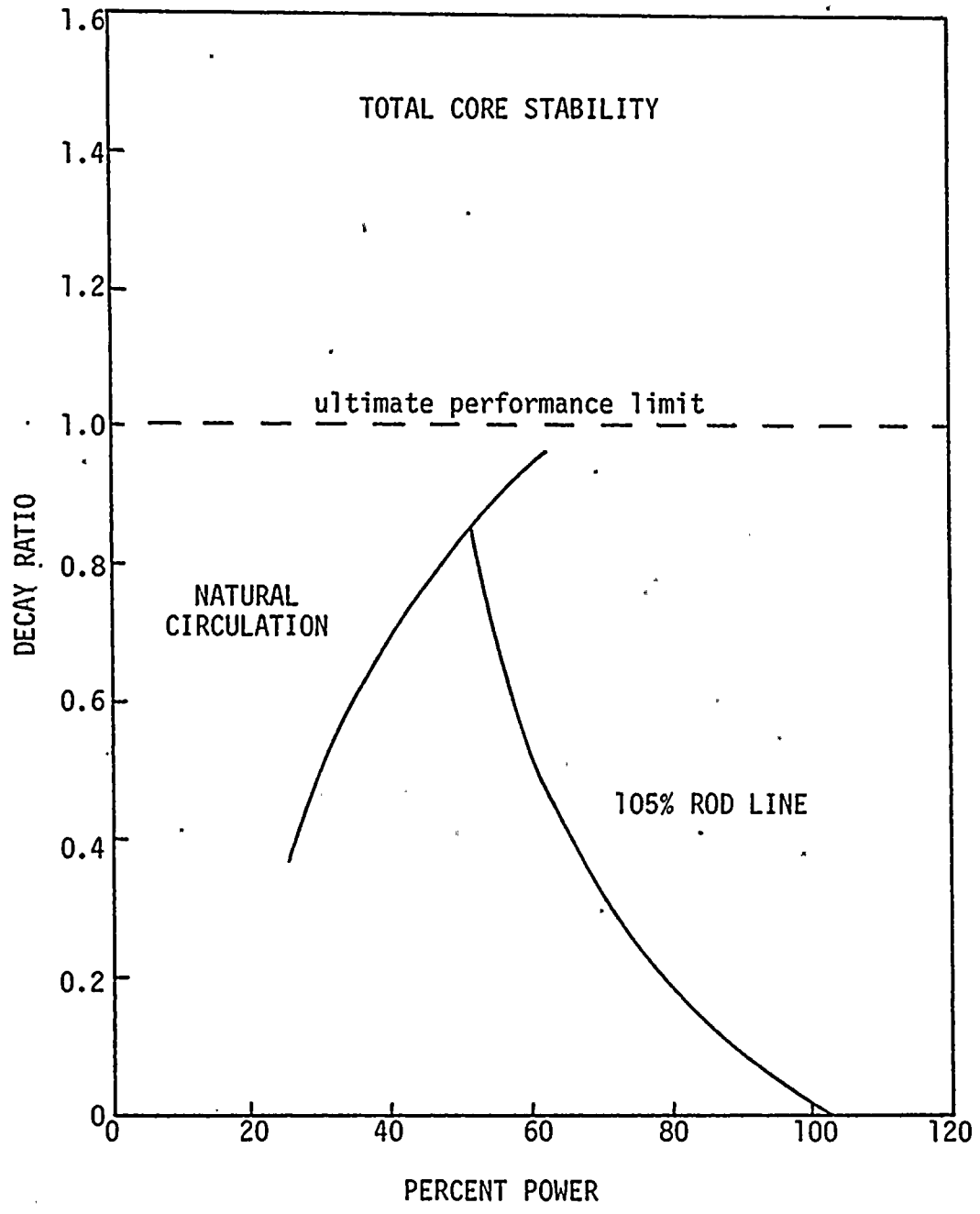
AUG 15 1983

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SSES PLA-1792
ER 100450 File 841-8
Mr. A. Schwencer

Attachments

cc: R. L. Perch - US NRC
S. Sun - US NRC



TOTAL CORE STABILITY

ultimate performance limit

NATURAL CIRCULATION

105% ROD LINE

PERCENT POWER

CORE REACTIVITY STABILITY