

TABLE 4.11.2.1.2-1

RADIOACTIVE GASEOUS WASTE SAMPLING AND ANALYSIS PROGRAM

Gaseous Release Type*	Sampling Frequency	Minimum Analysis Frequency	Type of Activity Analysis	Lower Limit of Detection (LLD) ^a ($\mu\text{Ci/ml}$)
A. Containment Purge	^P Each Purge ^b Grab Sample	^P Each Purge ^b	Principal Gamma Emitters ^g	1×10^{-4}
			H-3	1×10^{-6}
B. Reactor Building Vents, Turbine Building Vents, and SGTS	^M ^b Grab Sample	^M ^b	Principal Gamma Emitters ^{g-h}	1×10^{-4}
			H-3	1×10^{-6}
C. All Release Types as listed in A and B.	Continuous ^f	^W ^{c,d} Charcoal Sample	I-131	1×10^{-12}
	Continuous ^f	^W ^{c,d} Particulate Sample	Principal Gamma Emitters ^b (I-131, Others)	1×10^{-11}
	Continuous ^f	^Q Composite Particulate Sample	Gross Alpha	1×10^{-11}
	Continuous ^f	^Q Composite Particulate Sample	Sr-89, Sr-90	1×10^{-11}
	Continuous ^f	Noble Gas Monitor	Noble Gases Gross Beta or Gamma	1×10^{-6} (XE-133 equivalent)

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TABLE 4.11.2.1.2-1 (Continued)

TABLE NOTATION

- b. If the iodine or particulate monitoring channel(s) is(are) inoperative, analyses shall also be performed following shutdown, startup, or a THERMAL POWER change exceeding 15 percent of the RATED THERMAL POWER within a one hour period.
- c. Particulate and/or charcoal samples shall be analyzed when an alarm is received indicating rate of activity buildup exceeds 3 times normal
- d. Samples shall be changed at least once per 7 days and analyses shall be completed within 48 hours after changing, or after removal from sampler. If the iodine or particulate monitoring channel(s) is (are) inoperative, sampling shall also be performed at least once per 24 hours for at least 7 days following each shutdown, startup or THERMAL POWER change exceeding 15 percent of RATED THERMAL POWER in one hour and analyses completed within 48 hours of changing. When samples collected for 24 hours are analyzed, the corresponding LLDs may be increased by a factor of 10.
- e. (Deleted)
- f. The ratio of the sample flow rate to the sampled stream flow rate shall be known for the time period covered by each dose or dose rate calculation made in accordance with Specifications 3.11.2.1, 3.11.2.2 and 3.11.2.3.
- g. The principal gamma emitters for which the LLD specification applies exclusively are the following radionuclides: Kr-87, Kr-88, Xe-133, Xe-133m, Xe-135, Xe-135m and Xe-138 for gaseous emissions and Mn-54, Fe-59, Cu-58, Co-60, Zn-65, Mo-99, Cs-134, Cs-137, Ce-141 and Ce-144 for particulate emissions. This list does not mean that only these nuclides are to be considered. Other gamma peaks which are identifiable, together with those of the above nuclides, shall also be analyzed and reported in the Semiannual Radioactive Effluent and Release Report, pursuant to Specification 6.9.1.11.
- h. Under the provisions of footnote g above, only noble gases need to be considered.



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ATTACHMENT B

TABLE 3.7.6.5-1

FIRE HOSE STATIONS

<u>LOCATIONS</u>	<u>COLUMN</u>	<u>HOSE RACK NUMBER</u>
a. Control Structure		
E1. 697'-0"	L-26	1HR-171
E1. 697'-0"	L-32	2HR-171
E1. 714'-0"	L-26	1HR-162
E1. 714'-0"	L-31	2HR-162
E1. 729'-0"	L-25.9	1HR-158
E1. 729'-0"	L-32.1	2HR-158
E1. 754'-0"	L-26	1HR-136
E1. 754'-0"	L-32	2HR-136
E1. 771'-0"	L-26	1HR-125
E1. 771'-0"	L-31	2HR-125
b. Reactor Building		
E1. 645'-0"	R-29	1HR-271
E1. 645'-0"	P-20.6	1HR-272
E1. 645'-0"	U-22	1HR-273
E1. 645'-0"	R-37.4	2HR-271
E1. 645'-0"	U-30.5	2HR-272
E1. 645'-0"	R-30	2HR-273
E1. 670'-0"	Q-27.5	1HR-261
E1. 670'-0"	Q-29	1HR-262
E1. 670'-0"	T-22	1HR-263
E1. 683'-0"	Q-27.5	1HR-251
E1. 683'-0"	Q-20.6	1HR-252
E1. 683'-0"	T-22	1HR-253
E1. 719'-1"	Q-27.5	1HR-241
E1. 719'-1"	S-27.5	1HR-242
E1. 719'-1"	Q-20.6	1HR-243
E1. 719'-1"	T-20.6	1HR-244
E1. 719'-1"	T-23.5	1HR-245
E1. 749'-1"	S-27.5	1HR-231
E1. 749'-1"	Q-20.6	1HR-232
E1. 749'-1"	T-20.6	1HR-233
E1. 779'-1"	P-26.5	1HR-221
E1. 779'-1"	S-26.5	1HR-222
E1. 779'-1"	Q-22	1HR-223
E1. 779'-1"	U-20.6	1HR-224
E1. 799'-1"	T-23.3	1HR-211
E1. 818'-1"	P-26.5	1HR-201
E1. 818'-1"	U-26.5	1HR-202
E1. 818'-1"	Q-20.6	1HR-203



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