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SUBJECT: Forwards application for amend to License NPF-14, changing Tech Specs to incorporate final setpoint data developed during startup test program.

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## Pennsylvania Power & Light Company

Two North Ninth Street • Allentown, PA 18101 • 215 / 770-5151

Norman W. Curtis Vice President-Engineering & Construction-Nuclear 215/770-7501

APR 1 5 1983

Director of Nuclear Reactor Regulation Attention: Mr. A. Schwencer, Chief Licensing Branch No. 2 Division of Licensing U.S. Nuclear Regulatory Commission Washington, D.C. 20555

SUSQUEHANNA STEAM ELECTRIC STATION PROPOSED AMENDMENT 22 TO LICENSE NO. NPF 14 ER 100450 FILE 841-8 PLA-1622

Docket No. 50-387

Dear Mr. Schwencer:

8304180578 830415 PDR ADOCK 05000387

The purpose of this letter is to propose a change to the Susquehanna SES Unit 1 Technical Specifications.

Technical Specification Table 3.3.2-2, Trip Function 6a, is footnoted to indicate that a final setpoint will be developed during the Startup Test Program (see Attachment A). Startup Test 15.3, HPCI Rated Pressure Quick Starts to the Vessel, was approved on January 16, 1983, and PP&L is proposing to amend the Technical Specifications based on the data taken in that test.

General Electric recommends that the setpoint be based on greater than 272 but less than or equal to 300 percent of the maximum steady state flow rate through the HPCI steam line. The allowable value is then derived from the following equation provided by GE:

 $dp(trip) = dp(test) X \begin{bmatrix} \frac{3w(max)}{w(test)} \end{bmatrix}^2 X \frac{\rho(test)}{\rho(max)}$ 

This being the system design basis (which is consistent with IE Information Notice No. 82-16: HPCI/RCIC High Steam Flow Setpoints), the setpoint will be low enough to provide break detection in the HPCI steam supply line and initiate closure of the steam supply isolation valves, yet high enough to avoid spurious isolation trips resulting from pressure transients associated with quick starts of the system (Susquehanna has incorporated a time delay in the HPCI instrumentation system to avoid such trips).

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SSES PLA-1622 ER 100450 File 841-8 Mr. A. Schwencer

In order to assure ourselves that the 300 percent criteria used as the basis for HPCI isolation was within the envelope of the Susquehanna safety analyses, calculations were performed which indicate that the break area associated with 3 times maximum steady state HPCI steamline flow was approximately 0.1 square feet. GE has previously analyzed steamline breaks of this size and found that with the RCIC and CRD systems available, the top of active fuel remains covered by several feet.

With respect to the radiological consequences associated with the maximum break not isolated by the 300 percent value, our analysis shows that the expected dose would be enveloped by the dose associated with a design basis LOCA and therefore well within 10CFR100 limits.

Based on the information provided above, the proposed change will not jeopardize the safe operation of Susquehanna SES. The appropriate amendment fees pursuant to 10CFR170 are enclosed.

Very truly yours,

N. W. Curtis Vice President-Engineering & Construction-Nuclear

Enclosure

cc: R. L. Perch - USNRC

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