

PENNSYLVANIA POWER & LIGHT COMPANY
ALLEGHENY ELECTRIC COOPERATIVE, INC.
DOCKET NO. 50-387
SUSQUEHANNA STEAM ELECTRIC STATION, UNIT 1
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 8
License No. NPF-14

1. The Nuclear Regulatory Commission (the Commission or the NRC) having found that:
 - A. The application for an amendment filed by the Pennsylvania Power & Light Company dated October 22, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-14 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 8, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. PP&L shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

B302180029 830207
PDR ADCK 05000387
PDR

OFFICE▶
SURNAME▶
DATE▶

3. This amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

HS

A. Schwencer, Chief
Licensing Branch No. 2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: FEB 7 1983

*No legal objection
to terms of license
amendment*

AS

AD

OFFICE	DL:LB#2/PT	DL:LB#2/LA	OELD	DL:LB#2/BC	DL:AD/L		
SURNAME	RPerch:pt	EH015on	COTCHIN	ASchwencer	TNovak		
DATE	1/26/83	1/26/83	1/26/83	2/1/83	2/1/83		

14

28

ATTACHMENT TO LICENSE AMENDMENT NO. 8
FACILITY OPERATING LICENSE NO. NPF-14
DOCKET NO. 50-387

Replace the following pages of the Appendix "A" Technical Specifications with enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

REMOVE

3/4 6-3
3/4 6-4

3/4 8-17
3/4 8-18

3/4 11-17
3/4 11-18

INSERT

3/4 6-3
3/4 6-4

3/4 8-17
3/4 8-18

3/4 11-17
3/4 11-18

CONTAINMENT SYSTEMS

LIMITING CONDITION FOR OPERATION (Continued)

ACTION (Continued)

restore:

- a. The overall integrated leakage rate to less than or equal to $0.75 L_a$, and
- b. The combined leakage rate for all penetrations and all valves listed in Table 3.6.3-1, except for main steam line isolation valves*, main steam line drain valves* and valves which are hydrostatically leak tested per Table 3.6.3-1, subject to Type B and C tests to less than or equal to $0.60 L_a$, and
- c. The leakage rate to less than or equal to 46 scf per hour for all four main steam lines through the isolation valves, and
- d. The leakage rate to less than or equal to 1.2 scf per hour for any one main steam line drain valve, and
- e. The combined leakage rate for all containment isolation valves in hydrostatically tested lines which penetrate the primary containment to less than or equal to 3.3 gpm,

prior to increasing reactor coolant system temperature above 200°F.

SURVEILLANCE REQUIREMENTS

4.6.1.2 The primary containment leakage rates shall be demonstrated at the following test schedule and shall be determined in conformance with the criteria specified in Appendix J of 10 CFR 50 using the methods and provisions of ANSI N45.4 - 1972:

- a. Three Type A Overall Integrated Containment Leakage Rate tests shall be conducted at 40 ± 10 month intervals during shutdown at P_a , 45.0 psig, during each 10-year service period. The third test of each set shall be conducted during the shutdown for the 10-year plant inservice inspection.
- b. If any periodic Type A test fails to meet $.75 L_a$, the test schedule for subsequent Type A tests shall be reviewed and approved by the Commission. If two consecutive Type A tests fail to meet $.75 L_a$, a Type A test shall be performed at least every 18 months until two consecutive Type A tests meet $.75 L_a$, at which time the above test schedule may be resumed.
- c. The accuracy of each Type A test shall be verified by a supplemental test which:
 1. Confirms the accuracy of the test by verifying that the difference between the supplemental data and the Type A test data is within $0.25 L_a$.
 2. Has duration sufficient to establish accurately the change in leakage rate between the Type A test and the supplemental test.
 3. Requires the quantity of gas injected into the containment or bled from the containment during the supplemental test to be equivalent to at least 25 percent of the total measured leakage at P_a , 45.0 psig.

CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

- d. Type B and C tests shall be conducted with gas at P_a , 45.0 psig,* at intervals no greater than 24 months except for tests involving:
 - 1. Air locks,
 - 2. Main steam line isolation valves and main steam line drain valves,
 - 3. Containment isolation valves in hydrostatically tested lines which penetrate the primary containment, and
 - 4. Purge supply and exhaust isolation valves with resilient material seals.
- e. Air locks shall be tested and demonstrated OPERABLE per Surveillance Requirement 4.6.1.3.
- f. Main steam line isolation valves and main steam line drain valves shall be leak tested at least once per 18 months.
- g. Containment isolation valves in hydrostatically tested lines which penetrate the primary containment shall be leak tested at least once per 18 months.
- h. Purge supply and exhaust isolation valves with resilient material seals shall be tested and demonstrated OPERABLE per Surveillance Requirement 4.6.1.8.2.
- i. The provisions of Specification 4.0.2 are not applicable to 24 month or 40 ± 10 month surveillance intervals.

*Unless a hydraulic test is required per Table 3.6.3-1.

ELECTRICAL POWER SYSTEMS

3/4.8.3 ONSITE POWER DISTRIBUTION SYSTEMS

DISTRIBUTION - OPERATING

LIMITING CONDITION FOR OPERATION

3.8.3.1 The following power distribution system divisions shall be energized with tie breakers open both between redundant buses within the unit and between units at the same station:

a. A.C. power distribution:

1. Division I, consisting of:

- a) Load group Channel "A", consisting of:
 - 1) 4160 volt A.C. switchgear bus 1A201
 - 2) 480 volt A.C. load center 1B210
 - 3) 480 volt A.C. motor control center 0B516
- b) Load group Channel "C", consisting of:
 - 1) 4160 volt A.C. switchgear bus 1A203
 - 2) 480 volt A.C. load center 1B230
 - 3) 480 volt A.C. motor control center 0B536
- c) Load group 480 volt A.C. motor control centers 0B517, 0B136
1B216, 1B236
1B217, 1B237
1B219
- d) Load group 208/120 volt A.C. instrument panels 1Y216, 1Y236

2. Division II, consisting of:

- a) Load group Channel "B", consisting of:
 - 1) 4160 volt A.C. switchgear bus 1A202
 - 2) 480 volt A.C. load center 1B220
 - 3) 480 volt A.C. motor control center 0B526
- b) Load group Channel "D", consisting of:
 - 1) 4160 volt A.C. switchgear bus 1A204
 - 2) 480 volt A.C. load center 1B240
 - 3) 480 volt A.C. motor control center 0B546
- c) Load group 480 volt A.C. motor control centers 0B527, 0B146
1B226, 1B246
1B227, 1B247
1B229
- d) Load group 208/120 volt A.C. instrument panels 1Y226, 1Y246

b. D.C. power distribution:

1. Division I, consisting of:

- a) Load group Channel "A", consisting of:
 - 1) 125 volt DC buses 1D612, 1D614
 - 2) Fuse box 1D611
- b) Load group Channel "C", consisting of:
 - 1) 125 volt DC buses 1D632, 1D634
 - 2) Fuse box 1D631
- c) Load group "I", consisting of:
 - 1) 250 volt DC buses 1D652, 1D254
 - 2) Fuse box 1D651
- d) Load group "I", consisting of:
 - 1) \pm 24 volt DC buses 1D672
 - 2) Fuse box 1D671

ELECTRICAL POWER SYSTEMS

LIMITING CONDITION FOR OPERATION (Continued)

ACTION: (Continued)

D.C. power distribution: (Continued)

2. Division II, consisting of:

- | | |
|--|---------------------|
| a) Load group Channel "B" consisting of: | |
| 1) 125 volt DC buses | 1D622, 1D624 |
| 2) Fuse box | 1D621 |
| b) Load group Channel "D" consisting of: | |
| 1) 125 volt DC buses | 1D642, 1D644 |
| 2) Fuse box | 1D641 |
| c) Load group "II" consisting of: | |
| 1) 250 volt DC buses | 1D662, 1D264, 1D274 |
| 2) Fuse box | 1D661 |
| d) Load group "II" consisting of: | |
| 1) \pm 24 volt DC buses | 1D682 |
| 2) Fuse box | 1D681 |

\ APPLICABILITY: OPERATIONAL CONDITIONS 1, 2 and 3.

ACTION:

- a. With one of the above required A.C. distribution system load groups not energized, re-energize the load group within 8 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
- b. With one of the above required D.C. distribution system load groups not energized, re-energize the load group within 2 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

SURVEILLANCE REQUIREMENTS

4.8.3.1 Each of the above required power distribution system load groups shall be determined energized at least once per 7 days by verifying correct breaker alignment and voltage on the busses/MCCs/panels.

RADIOACTIVE EFFLUENTS

EXPLOSIVE GAS MIXTURE

LIMITING CONDITION FOR OPERATION

3.11.2.6 The concentration of hydrogen or oxygen in the main condenser offgas treatment system shall be limited to less than or equal to 4% by volume.

APPLICABILITY: Whenever the main condenser air ejector (evacuation) system is in operation.

ACTION:

- a. With the concentration of hydrogen in the main condenser offgas treatment system exceeding the limit, restore the concentration to within the limit within 48 hours.
- b. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.11.2.6 The concentration of hydrogen in the main condenser offgas treatment system shall be determined to be within the above limits by continuously monitoring the waste gases in the main-condenser offgas treatment system with the hydrogen monitors required OPERABLE by Table 3.3.7.11-1 of Specification 3.3.7.11.

RADIOACTIVE EFFLUENTS

MAIN CONDENSER

LIMITING CONDITION FOR OPERATION

3.11.2.7 The radioactivity rate of the noble gases Kr-85m, Kr-87, Kr-88, Xe-133, Xe-135, and Xe-138 measured at the motive steam jet condenser discharge shall be limited to less than or equal to 330 millicuries/second.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2* and 3*.

ACTION:

With the gross radioactivity rate of the specified noble gases at the motive steam jet condenser discharge exceeding 330 millicuries/second, restore the gross radioactivity rate to within its limit within 72 hours or be in at least HOT STANDBY within the next 12 hours.

SURVEILLANCE REQUIREMENTS

4.11.2.7.1 The radioactivity rate of noble gases at the motive steam jet condenser discharge shall be continuously monitored in accordance with Specification 3.3.7.11.

4.11.2.7.2 The gross radioactivity rate of the specified noble gases from the motive steam jet condenser discharge shall be determined to be within the limits of Specification 3.11.2.7 at the following frequencies by performing an isotopic analysis of a representative sample of gases taken at the discharge:

- a. At least once per 31 days.
- b. Within 4 hours following an increase, as indicated by the Condenser Offgas Pre-Treatment Radioactivity Monitor, of greater than 50%, after factoring out increases due to changes in THERMAL POWER level, in the nominal steady state fission gas release from the primary coolant.

*When the main condenser air ejector is in operation.