

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 8208310180 DOC. DATE: 82/08/25 NOTARIZED: NO DOCKET #
 FACIL: 50-387 Susquehanna Steam Electric Station, Unit 1, Pennsylvania 05000387
 AUTH. NAME: CURTIS, N.W. AUTHOR AFFILIATION: Pennsylvania Power & Light Co.
 RECIP. NAME: SCHWENGER, A. RECIPIENT AFFILIATION: Licensing Branch '2

SUBJECT: Advises of recheck/review of diesel generator bldg, reactor/control bldg (vertical model) & ESSW pumphouse. Calculations do not contain errors. Original design basis adequate. Response satisfies conditions to review models by 820825.

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INTERNAL:	ELD/HDS4	1	0	IE FILE		1			1
	IE/DEP EPDS 35	1	1	IE/DEP/EPLB 36		3			3
	NRR/DE/AEAB	1	0	NRR/DE/CEB 11		1			1
	NRR/DE/EQB 13	3	3	NRR/DE/GB 28		2			2
	NRR/DE/HGEB 30	2	2	NRR/DE/MEB 18		1			1
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	NRR/DHFS/HFEB40	1	1	NRR/DHFS/LQB 32		1			1
	NRR/DHFS/OLB 34	1	1	NRR/DHFS/PTRB20		1			1
	NRR/DSI/AEB 26	1	1	NRR/DSI/ASB 27		1			1
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	NRR/DSI/RSB 23	1	1	NRR/DST/LGB 33		1			1
	<u>REG FILE</u> 04	1	1	RGN1		2			2
	RM/DDAMI/MIB	1	0						
EXTERNAL:	ACRS 41	10	10	BNL (AMDTs ONLY)		1			1
	DMB/DSS (AMDTs)	1	1	FEMA-REP DIV 39		1			1
	LPDR 03	2	2	NRC PDR 02		1			1
	NSIC 05	1	1	NTIS		1			1

DB

MISSION NBR: 850831080 DOC DATE: 85\08\25 MATURITY: 0
LIC: 50-387 Susquehanna Steam Electric Station, Unit 1, Pennsylvania
AUTHOR AFFILIATION: Pennsylvania Power & Light Co.
RECIP NAME: LICENSING BRANCH 5
RECIP AFFILIATION:

ADVISOR: ADVISOR OF PROCEEDINGS/PROVIDER OF DIESEL GENERATOR DIESECTORY CONTROL BID (VERTICAL MODEL) & ESRW PUMPHOUSE CALCULATIONS DO NOT CONTAIN ERRORS. ORIGINAL DESIGN DATA ADEQUATE. RESPONSE STATISTICS CONDITIONS TO REVIEW MODELS BY 850825.

TITLE: LICENSING SUBMITTAL: PSARPASR Amdts & Related Correspondence
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LEAD EPDS	1	LEAD EPDS	36	WFL (AMTS ONLY)	1
ARRDEV\VEAB	0	ARRDEV\VEAB	11		
ARRDEV\VEAB	3	ARRDEV\VEAB	58		
ARRDEV\NGEB	5	ARRDEV\VEAB	18		
ARRDEV\NGEB	1	ARRDEV\VEAB	51		
ARRDEV\MTB	1	ARRDEV\VEAB	52		
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Pennsylvania Power & Light Company

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Norman W. Curtis
Vice President-Engineering & Construction-Nuclear
215 / 770-5381

AUG 25 1982

Mr. A. Schwencer, Chief
Licensing Branch No. 2
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

SUSQUEHANNA STEAM ELECTRIC STATION
NPF-14, LICENSE CONDITION C(11)
ER 100450/100508 FILE 842-05, 170
PLA-1255

DOCKET NO. 50-387

Dear Mr. Schwencer,

As committed to in our letter (PLA-1184 dated July 13, 1982) and subsequently required as license condition number C(11) to our operating license (NPF-14) we have performed a recheck of all calculations used in development of stick models for the Reactor/Control Building (vertical model), Diesel Generator Building and the ESSW Pumphouse. The results of the rechecking efforts for the Reactor/Control Building (horizontal models) and Containment have been previously transmitted to you in PLA-1184 and PLA-1188, respectively.

The modeling calculations were rechecked by Bechtel Power Corporation. Pennsylvania Power and Light Company reviewed the rechecked calculations to assure that the recheck was completed and documented as required. All calculations associated with calculating masses, section properties and spring stiffnesses used in the stick models were rechecked to assure that these calculations do not contain errors to which the models would be sensitive. In addition, this recheck also covered the checking of the transposition of all calculation results to computer input.

The recheck/review of the Diesel Generator Building, the Reactor/Control Building (vertical model) and the ESSW Pumphouse showed that the calculations do not contain errors to which the models would be sensitive. However, during the recheck/review of the Reactor/Control Building's vertical model, some of the original modeling assumptions and representations were determined to be worthy of further investigation. To perform these investigations a study model was developed.

As can be expected slight changes in response spectra did result at certain elevations in the study model. The global affect, however, was small as the major stick behavior did not change appreciably.

BOO!

8208310180 820825
PDR ADOCK 05000387
PDR



AUG 25 1982

ER 100450/100508

PLA-1255
File 842-05,170

Page 2

To further assure the adequacy of the plant, the safety related systems and components were assessed against the study model spectra. The assessment program employed the same methodology as that used in the previous assessment programs (see PLA-1147 for a description of the program.) The assessment program found all safety related systems and components to be adequate. This conclusion reaffirms the adequacy of our original design basis.

The response presented herein satisfies the condition imposed on us in the operating license to review the Diesel Generator Building, Reactor/Control Building and ESSW Pumphouse models by August 25, 1982.

Very truly yours,

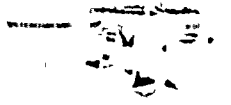


N. W. Curtis

Vice President-Engineering and Construction-Nuclear

TAG/JDV:mcr

cc: R. L. Perch NRC



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