

NORMAN W. CURTIS
Vice President-Engineering & Construction
821-5381

September 5, 1979

Mr. Boyce H. Grier
Director, Region I
U. S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, Pennsylvania 19406

SUSQUEHANNA STEAM ELECTRIC STATION
INTERIM REPORT OF A DEFICIENCY
IN PIPE SUPPORT DESIGN DRAWINGS
RELEASED FOR FABRICATION AND INSTALLATION
ERs 100450/100508 FILE 840-4
PLA-391

Dear Mr. Grier:

This serves to confirm information provided to R. Gallo, NRC, Reactor Inspector by A. R. Sabol, PP&L, on August 2, 1979 regarding the subject deficiency which is deemed reportable under 10 CFR 50.55(e).

The deficiency involves discrepancies between the Bechtel Plant Design Group's finalized stress analyses calculations and the loads which were used in the design of pipe support detail drawings which were fabricated and released for installation during the construction of the plant. The released pipe support detail drawings did not have factored into them revised piping system stress values which had been developed subsequent to the initial piping system stress calculations.

All of the safety implications are not known at this time; however, our best estimate is that there will be approximately 1000 pipe supports which may require design changes. Had the deficiencies gone undetected, it is anticipated that some supports could have been overstressed beyond allowable design limits by an order of 100 percent or greater.

The condition was first detected by Bechtel engineering personnel during preparations for piping systems stress walkdowns in the first quarter of 1979. At that time, a review performed on the basis of a random sample indicated that the problem was not generic or widespread. The problem appeared to have been confined to calculations which had been performed prior to mid-1976 and subsequently revised.

September 5, 1979

Subsequently, in preparation for turnover of the feedwater system to startup, a relatively large number of pipe support details, approximately 40 out of 276, which were found to be discrepant. Therefore, in order to facilitate as orderly and complete as possible turnover of systems for startup, a thorough verification program has been instituted. The verification program, which instituted following discovery of the condition, will cover all force, moment calculations and their related pipe support details.

On July 25, 1979, Bechtel QA identified the condition in its MCAR-1-36 requesting that Bechtel engineering evaluate the condition for reportability. Based upon Bechtel engineering's conclusion that the condition is reportable, a final report to MCAR-1-36 will be submitted to PP&L by December 28, 1979. Following, PP&L will submit its final report of said deficiency to the Commission by March 1, 1980.

Very truly yours,



N. W. Curtis
Vice President-Engineering & Construction

ARS:mcb

cc: Mr. Robert M. Galló
U. S. Nuclear Regulatory Commission
P. O. Box 52
Shickshinny, Pennsylvania 18655

Mr. Victor Stello (15)
Director-Office of Inspection & Enforcement
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Mr. G. McDonald
Office of Management Information & Program Control
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555