U.S. NUCLEAR REGULATORY COMMISSION OFFICE OF INSPECTION AND ENFORCEMENT

Region I

Report No	78-22					
Docket No	50-387					
License No.	CPPR-101	Priority _	-	Cate	egory	A
Licensee: _	Pennsylvania P	ower and Light	_Company		•	
_2	North Ninth	Street		4		
_£	Allentown, Pen	nsylvania 1810	01			
Facility Name	e: <u>Susquehan</u>	na Steam Elect	ric Station.	, Unit 1		
Inspection a	t: Berwick,	Pennsylvania ·				•
Inspection co	onducted: De	cember 12-14,	1978			,
Inspectors:	So, On	Calton.			12-	28-78
	G. Walton, Re	actor Inspector	r	ě,		e signed
	IL W	alton for			12-2	28-78
	• • •	Reactor Inspec	tor		date	signed
			·	•	date	e signed
Approved by:	SI Show	Un			1/41	79
		r, Acting Chie	f, Engineeri	ing	date	signed
		ection No. 1,			•	

Inspection Summary:

Inspection on December 12-14, 1978 (Report No. 50-387/78-22)

Areas Inspected: Routine, unannounced inspection by regional based inspectors of component storage, installation of incore housings in reactor pressure vessel, welding of safety-related piping and preservice inspection activities. The inspection involved 46 inspector-hours onsite by two NRC inspectors.

Results: No items of noncompliance or deviations were identified.



DETAILS

1. Persons Contacted

Pennsylvania Power and Light Company

- L. Ballard, Site QA Engineer
- *R. Beckley, Site QA Engineer
- *S. Denson, Assistant Project Construction Manager
- E. Carroll, Site QA Engineer
- *J. Green, Resident Nuclear QA Engineer
- *A. Male, Assistant Manager Nuclear Plant Engineering
- *T. Oldenhage, Resident Engineer
- *R. Shovlin, Assistant Project Director

United States Nuclear Regulatory Commission

*R. Gallo, Resident Reactor Inspector

Bechtel Corporation

- *M. Drucker, Lead QA Engineer
- H. Oman, Lead Storage and Maintenance Engineer
- K. Kutner, Warehouse Supervisor
- *K. Stout, Assistant Project Field QC Engineer
- A. Kenjura, Site QA Engineer
- T. Seymour, Site QA Engineer
- L. Price, Engineer
- *C. Turnbow, F. C. M.
- *J. O'Sullivan, Assistant P. F. E.
- *A. Spielhagen, P. F. E.
- *M. Boone, L. F. W. E.

General Electric Company

- F. Forsythe, I&SE Reactor Internals Site Manager
- D. Hess, Site QA Supervisor
- *T. LeVasseur, Site QC Representative
 - R. Pulsifer, Mechanical Engineer
- . G. Gustafson, Site QA Manager
- * denotes those present at the exit interview

2. Plant Tour

The inspectors observed work activities in progress, completed work, and plant status in several areas of the plant during a plant status inspection of the plant. The inspectors examined work items for any obvious defects or noncompliance with regulatory requirements. Where more detailed inspection of an area was conducted, the inspection scope and findings are described in other paragraphs of the report.

Reactor Coolant Pressure Boundary Piping (Welding)

The following listed activities were examined and verified that they were being performed in accordance with ASME Section III and IX Codes, PSAR Appendix D and the Bechtel Quality Assurance Manual.

Recirculate Line VRR-B31-1 FW-A1, Mt1. SA 240 Type 304 to SA 336 F8

The inspector verified weld card, welding procedure P8-AT-AG-R2, welder qualifications (P1A1, P6A9, PZ5 and P7K), quality control documentation, weld appearance, welding variables, and non-destructive testing activities as appropriate.

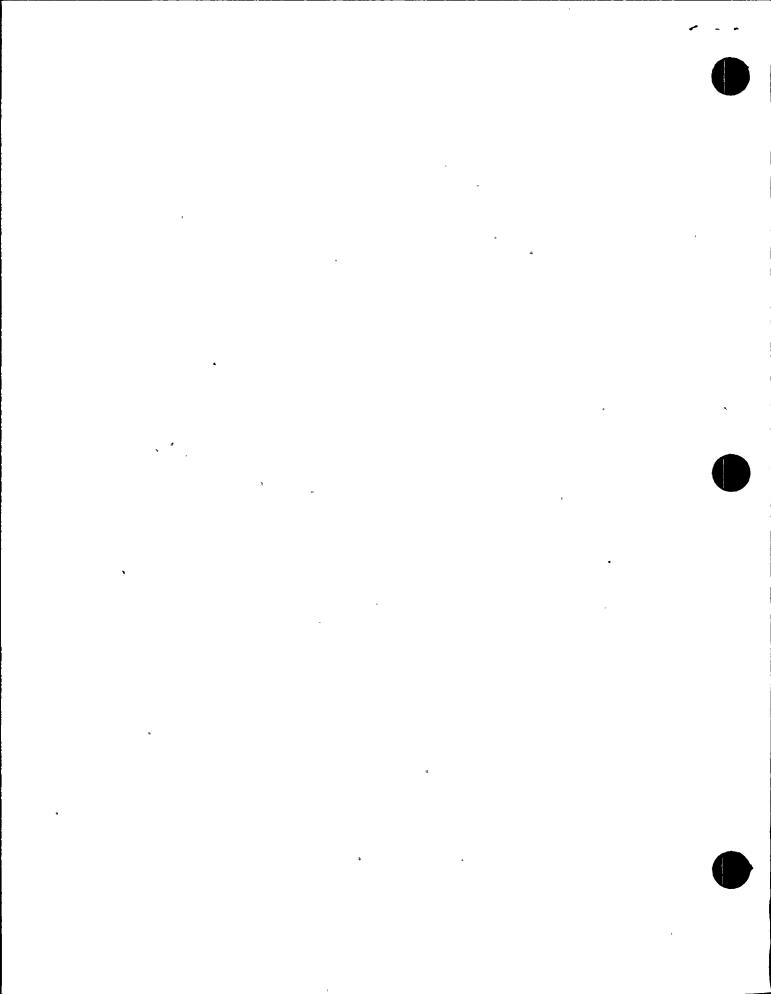
No items of noncompliance were identified.

4. Reactor Vessel In-Core Housing Installation

The following listed activities were examined for the installation of in-core housings in the reactor pressure vessel.

- a. Observation of welding
- b. Welder qualification
- c. Liquid penetrant examination results
- d. Review of process controls, hold points and calibration of measuring equipment

No items of noncompliance were identified.



5. Licensee Action on Previous Inspection Findings

(Open) Unresolved Item (387/78-16-03): The inspector requested final examination and evaluation results relative to the volumetric examination of the dissimilar metal welds in the recirculation nozzles. Cognizant licensee personnel indicated that the final examination report had not been issued, but would be available on January 16, 1979. The inspector's audit of examination data available during the inspection included radiographic data of the N2J nozzle safe-end to extension piece weld. The radiographic interpretation sheet for this nozzle indicated that the level III film interpreter had rejected this weld. The inspector also observed piping being fit-up to the recirculation nozzles in question. The licensee representatives stated that the evaluation was ongoing and their position would be determined by the results.

This item remains unresolved.

6. Safety-Related Components Observation of Work and Work Activities

The inspector audited procedures to control the storage, handling and protection of those pressure retaining components which will become part of the reactor pressure boundary. The inspector also observed the implementation of portions of these procedures and reviewed associated records.

These audits and observations were made to determine if the established program and its implementation were consistent with the requirements of Appendix D of the Facility PSAR.

a. Storage, Handling and Protection

The inspector toured safety-related equipment storage areas which included: 1) the on-site warehouse; 2) Pittston Warehouse; 3) plant drywell; and, 4) reactor building. The tour was conducted to determine whether storage requirements were being met relative to: 1) protection; 2) identification; 3) identification of nonconforming conditions; 4) lubrication; 5) preservative/heaters; and, 6) maintenance/surveillance activities.

The inspector observed the storage warehouse to be weathertight, heated enclosures constructed of steel with concrete. floors which contain an automatic sprinkler fire suppression system. The storage environment and protection afforded by these warehouses met or exceeded the equipment supplier's requirements. Additionally, the inspector observed the storage/protection provided for the recirculation system pumps and isolation valves, main steam isolation valves, residual heat removal pumps, core spray pumps and core spray pump barrels.

The inspectors found that the storage/protection provided the above equipment to be consistent with the requirements of the applicable procedures/specifications, except for the core spray pump barrels. These barrels were stored in-place and were filled with water. This condition was identified by the licensee during this inspection. Corrective action to preclude and mitigate this storage problem were in progress. The inspector stated these actions by the licensee's storage contractor were responsive to the conditions observed. The inspector had no further questions regarding this item at this time.

No items of noncompliance were identified by the inspector.

b. Work Procedures/Specifications

The inspector examined installation drawings and specifications associated with the recirculation pumps. These pumps were installed in position on elevation 704' of the reactor building. The manufacturer's installation specification IE-3429-6 and the contractor's drawing M-242, Revision 10 were reviewed. The inspector determined these documents to be applicable for this equipment. Additionally, the inspector audited inspection records associated with pump volute installation. This review indicated that the installation was performed in accordance with the appropriate drawings and specification.

No items of noncompliance were identified by the inspector.

7. Unresolved Item

Unresolved items are matters about which more information is required in order to ascertain whether they are acceptable items, items of non-compliance or deviations. An unresolved item disclosed during a previous inspection and reviewed during this inspection is discussed in Paragraph 5.

8. Exit Interview

The inspector met with the licensee representativies (denoted in paragraph 1) at the conclusion of the inspection. The inspectors summarized the purpose and scope of the inspection.

