NuScaleDCRaisPEm Resource

From: Cranston, Gregory
Sent: Tuesday, May 16, 2017 10:25 AM
To: RAI@nuscalepower.com
Cc: NuScaleDCRaisPEm Resource; Lee, Samuel; Chowdhury, Prosanta; Karas, Rebecca; Drzewiecki, Timothy; Bavol, Bruce
Subject: RE: Request for Additional Information No. 23, RAI 8772
Attachments: Request for Additional Information No. 23 (eRAI No. 8772).pdf

Attached please find NRC staff's request for additional information concerning review of the NuScale Design Certification Application.

Please submit your response within 60 days of the date of this RAI to the NRC Document Control Desk.

If you have any questions, please contact me.

Thank you.

Gregory Cranston, Senior Project Manager
Licensing Branch 1 (NuScale)
Division of New Reactor Licensing
Office of New Reactors
U.S. Nuclear Regulatory Commission
301-415-0546

Hearing Identifier: NuScale_SMR_DC_RAI_Public
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From: Cranston, Gregory

Created By: Gregory.Cranston@nrc.gov

Recipients:
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Request for Additional Information No. 23 (eRAI No. 8772)

Issue Date: 05/16/2017
Application Title: NuScale Standard Design Certification - 52-048
Operating Company: NuScale Power, LLC
Docket No. 52-048
Review Section: 04.03 - Nuclear Design
Application Section: 4.3

QUESTIONS

04.03-1

10 CFR 50.36(c)(2)(ii)(B) requires that a technical specification limiting condition for operation (LCO) be established for a "process variable, design feature, or operating restriction that is an initial condition of a design basis accident or transient analysis that either assumes the failure of or presents a challenge to the integrity of a fission product barrier." NuScale generic technical specifications (GTS) contain several LCOs that reference limits specified in the CORE OPERATING LIMITS REPORT (COLR), which is a defined term in GTS Section 1.1, "Definitions," and which is specified in GTS Subsection 5.6.3. Paragraph b of Subsection 5.6.3 states that, "The analytical methods used to determine the core operating limits shall be those previously reviewed and approved by the NRC, specifically those described in the following documents: …" The analytical method documents listed in paragraph b only include the Reload Safety Evaluation Methodology and the Nuclear Analysis Codes and Methods Qualification topical report, TR-0616-48793-NP. The listed Reload Safety Evaluation Methodology document is a placeholder for a document that has not yet been submitted for NRC review, and the basis for determining NuScale Power MODULE operating restrictions is outside the scope of the Nuclear Analysis Codes and Methods Qualification topical report. The NRC staff relies upon the LCOs to establish a finding that the plant will be operated within the bounds of the safety analyses, and the LCOs that reference the COLR are not supported by an established methodology. Accordingly, the NRC staff requests that NuScale update the FSAR, either directly or by reference, to provide information describing the methodology and technical basis for selecting the core operating limits that are contained within the COLR.