



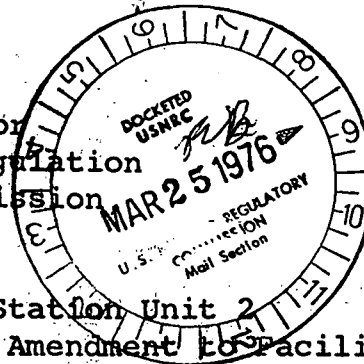
Commonwealth Edison
One First National Plaza, Chicago, Illinois
Address Reply to: Post Office Box 767
Chicago, Illinois 60690

Regulatory Docket File

March 15, 1976



Mr. Benard C. Rusche, Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555



Subject: Dresden Station Unit 2
Proposed Amendment to Facility License
No. DPR-19 and Appendix A Technical
Specifications, NRC Dkt. 50-237

Dear Mr. Rusche:

Pursuant to 10 CFR 50.59, Commonwealth Edison proposes to amend facility license DPR-19 and Appendix A Technical Specifications. The changes are shown on enclosed page 3 of the license and pages 81B, 81C, 81C-1, and 81C-2 of Appendix A Technical Specifications.

The license amendment required for the operation of Dresden Unit 2 Reload 2 (Cycle 5) is the modification of Section 3, "Restrictions". This revision relates the scram reactivity restriction to the present criteria but without reference to the cycle. This restriction will be consistent with that which applies to Quad-Cities Unit 2 DPR-30 and Dresden Unit 3 DPR-25.

In addition, it is proposed to delete the "Reports" and "Records" paragraph of Section 3 of the license. The license requirement to comply with the Technical Specification is already contained in paragraph 3B. This change is consistent with the Quad-Cities licenses DPR-29 and DPR-30.

The Technical Specifications change required is the inclusion of the MAPLHGR curve for 8x8, 2.62 w/o fuel which is introduced into the core for the first time. Technical Specification pages 81C and 81C-1 include not only the 8x8, 2.62 w/o MAPLHGR curves, but those for other applicable fuel installed in Dresden Unit 2. These curves are the same as those submitted in Dresden Station Special Report No. 40, Supplement B and are not the same as those presently in the Technical Specifications. Enclosure (1) provides the technical support

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Mr. Benard C. Rusche

- 2 -

March 15, 1976

for the reload changes.

Also included with the Technical Specification changes are those requested for APLHGR and LHGR in D. L. Ziemann's letter to R. L. Bolger dated March 1, 1976.

For your schedule, projected startup date for Dresden Unit 2 is May 31, 1976.

This change has received Onsite and Offsite approval.

Three (3) signed originals and 37 copies are enclosed for your use.

Please address any questions to this office.

Very truly yours,



R. L. Bolger
Assistant Vice President

- Enclosure (1) General Electric Boiling Water Reactor Reload No. 2 Licensing Submittal for Dresden Nuclear Power Station Unit 2.
- (2) Proposed license and Technical Specification changes.

SUBSCRIBED and SWORN to
before me this 15th day
of March, 1976.

Nancy M. Hollingsworth
Notary Public

My Commission Expires September 24, 1978

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications, as revised.

C. Restrictions

Beyond the point in the fuel cycle at which the reactivity reduction rate during a scram is less than that of Curve B in Figure 1 of "Supplement B to Dresden Station Special Report 29," dated March 29, 1974, operation of the reactor shall not exceed the core thermal power versus flow conditions defined by the "Nominal Expected 90% Flow Control Line" on Figure 2.1-3 of the Commonwealth Edison letter (J. S. Abel to Benard C. Rusche) dated June 24, 1975.

Beyond the point in the fuel cycle at which the reactivity reduction rate during a scram is less than that of end-of-cycle curve on Figure 1-1 of the Commonwealth Edison letter (J. S. Abel to D. L. Zicmann) dated February 27, 1975, operation of the reactor is not authorized.

D. For the purpose of repairing a crack in the recirculation bypass line in the "A" loop, the licensee may perform the repair program as described in a report entitled "Commonwealth Edison Company Dresden Station 2A Recirculation Pump 4" Equalizing Line Repair Program" transmitted by letter dated September 23, 1974.

4. This license is effective as of the date of issuance and shall expire eighteen (18) months from said date, unless extended for good cause shown, or upon the earlier issuance of a superseding operating license.

FOR THE ATOMIC ENERGY COMMISSION

Attachment:

Appendix A - Technical Specifications

Date of Issuance: DEC 22 1969

Peter A. Morris

Peter A. Morris, Director
Division of Reactor Licensing

3.5 LIMITING CONDITION FOR OPERATION

I. Average Planar LHGR

During steady state power operation, the average linear heat generation rate (LHGR) of all the rods in any fuel assembly, as a function of average planar exposure, at any axial location, shall not exceed the maximum average planar LHGR shown in Figures 3.5.1-A or 3.5.1-B dependent on fuel type.

If at any time during operation it is determined by normal surveillance that the limiting value for APLHGR is being exceeded, action shall be initiated within 15 minutes to restore operation to within the prescribed limits. If the APLHGR is not returned to within the prescribed limits within two (2) hours, the reactor shall be brought to the Cold Shutdown condition within 36 hours. Surveillance and corresponding action shall continue until reactor operation is within the prescribed limits.

4.5 SURVEILLANCE REQUIREMENT

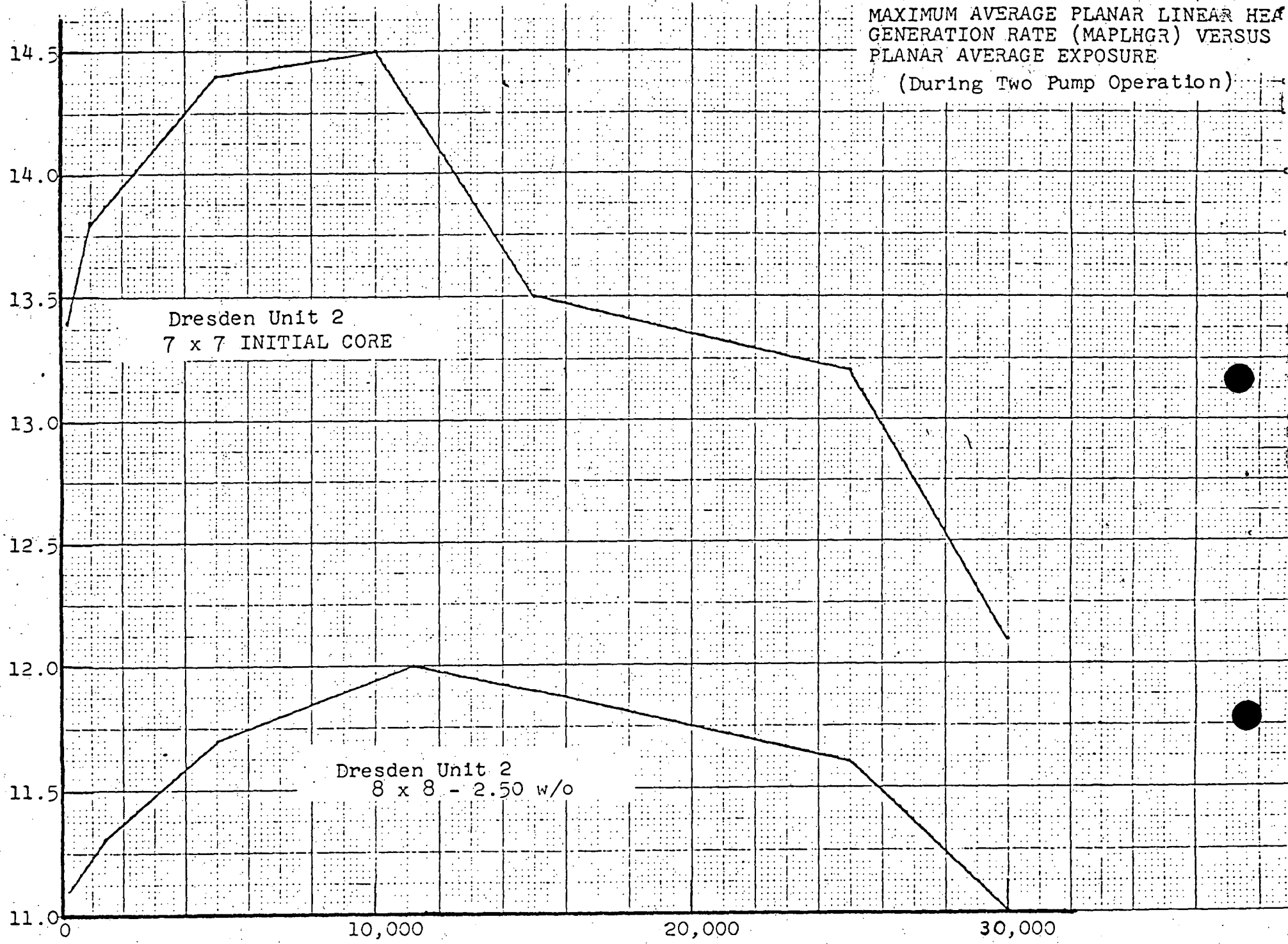
I. Average Planar LHGR

The APLHGR for each type of fuel as a function of average planar exposure shall be determined daily during reactor operation at $\geq 25\%$ rated thermal power.

MAXIMUM AVERAGE PLANAR LINEAR HEAT
GENERATION RATE (MAPLHGR) VERSUS
PLANAR AVERAGE EXPOSURE

(During Two Pump Operation)

Maximum Average Planar LHGR (kw/ft)



Dresden Unit 2
7 x 7 INITIAL CORE

Dresden Unit 2
8 x 8 - 2.50 w/o

PLANAR AVERAGE EXPOSURE (MWD/T)
FIGURE 3.5.1. A

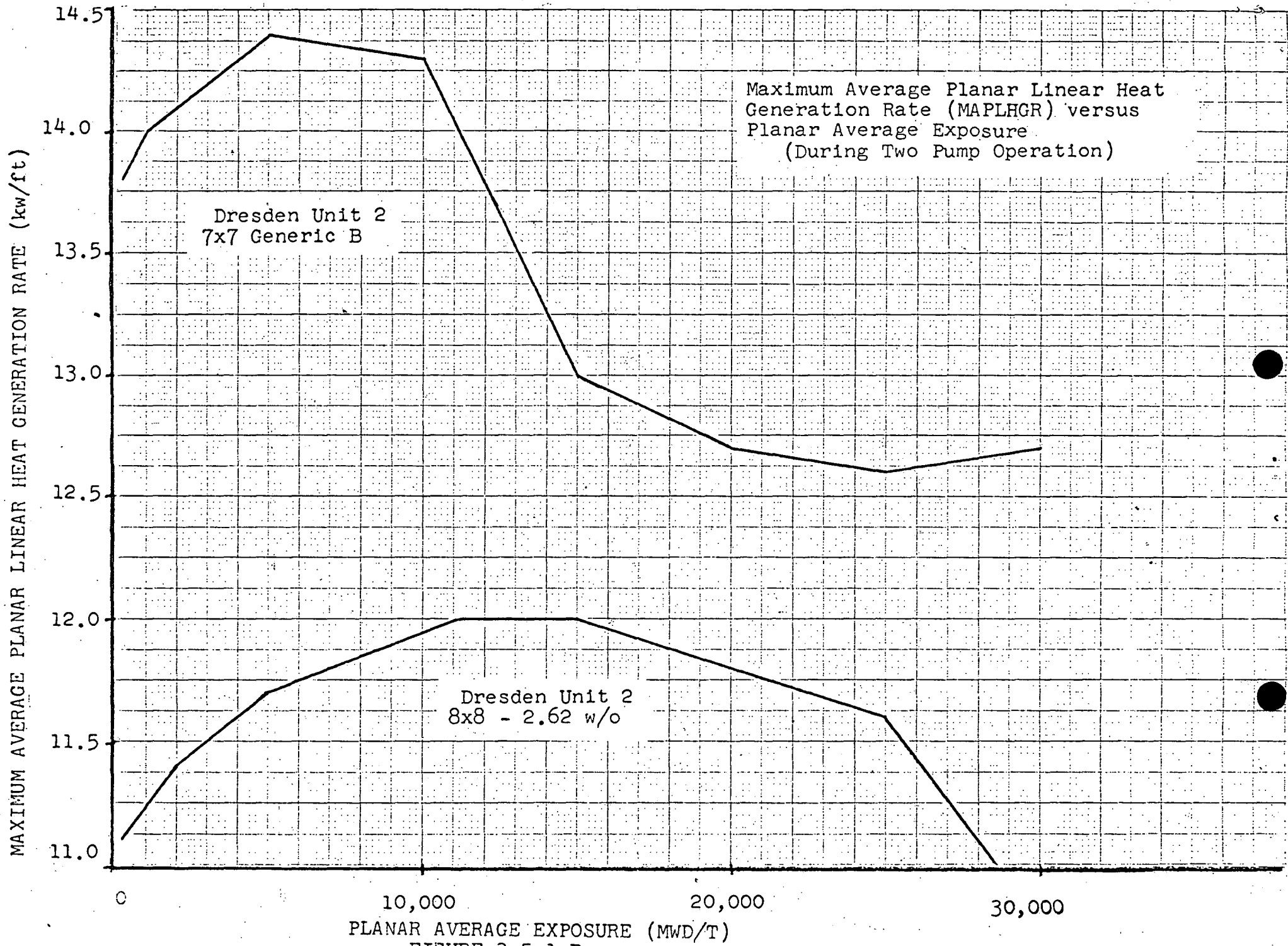


FIGURE 3.5.1.B

LIMITING CONDITION FOR OPERATION

J. Local LHGR

During steady state power operation, the linear heat generation rate (LHGR) of any rod in any fuel assembly at any axial location shall not exceed the maximum allowable LHGR as calculated by the following equation.

$$\text{LHGR}_{\text{max}} < \text{LHGR}_{\text{d}} \left[1 - \left(\frac{\Delta P}{P} \right)_{\text{max}} \left(\frac{L}{L_T} \right) \right]$$

- LHGR_d - Design LHGR
 = 17.5 kw/ft, 7x7 fuel assemblies
 = 13.4 kw/ft, 8x8 fuel assemblies

- $\left(\frac{\Delta P}{P} \right)_{\text{max}}$ - Maximum power spiking penalty
 - .037 initial core fuel
 - .026 reload 1, 7x7 fuel
 - .022 8x8 fuel

L_T - Total Core Length - 12 ft.

L - Axial distance from bottom of core

If at any time during operation it is determined by normal surveillance that the limiting value for LHGR is being exceeded, action shall be initiated within 15 minutes to restore operation to within the proscribed limits. If the LHGR is not returned to within the proscribed limits within two (2) hours, the reactor shall be brought to the Cold Shutdown condition within 36 hours. Surveillance and corresponding action shall continue until reactor operation is within the proscribed limits.

SURVEILLANCE REQUIREMENT

J. Local LHGR

The LHGR as a function of core height shall be checked daily during reactor operation at $\geq 25\%$ rated thermal power.