

August 2, 2017

MEMORANDUM TO: Dennis C. Morey, Chief
Licensing Processes Branch
Division of Policy and Rulemaking
Office of Nuclear Reactor Regulation

FROM: Brian J. Benney, Senior Project Manager /RA/
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SUBJECT: SUMMARY OF APRIL 11, 2017, CLOSED MEETING WITH THE
PRESSURIZED WATER REACTOR OWNERS GROUP TO
DISCUSS INFORMATION RELATED TO THE PWROG
PROJECT PA-ASC-1420, "CONTINUED USE OF
WCAP-10325-P-A, WESTINGHOUSE LOCA M+E"

On April 11, 2017, a closed meeting was held between the U.S. Nuclear Regulatory Commission (NRC) staff and the Pressurized Water Reactor Owners Group (PWROG). The purpose of this meeting was for the PWROG to discuss information related to the PWROG Project PA-ASC-1420, "Continued use of WCAP-10325-P-A, Westinghouse LOCA M+E Methodology."

NRC staff met with the PWROG on April 11, 2017, to discuss their proposed approach, as outlined in their letter dated March 17, 2017 (Agencywide Documents Access and Management System Package Accession No. ML17081A081), to resolve the material issue (stainless steel heat capacity) addressed in Westinghouse Infogram 14-1. Infogram 14-1 identified that the stainless steel heat capacity used for reactor coolant system (RCS) metal in WCAP-10325-P-A was lower than the ASME heat capacity (this could result in a lower containment peak accident pressure). NRC discussions with the PWROG had begun last year during a June 1, 2016, meeting on the proposed technical approach to resolve the heat capacity issue via a topical report (TR). NRC staff conveyed its concerns about the approach during the June meeting, which was further discussed during a telecom in the November 2016 timeframe. The PWROG's purpose at the April 11th meeting was to walk through the technical approach, address staff's technical concerns, and discuss the licensing approach with NRC staff.

The following highlights the discussions during the April 11th meeting:

- The TR would only be applicable to pressurized water reactors (PWRs) with large dry containments and not ice-condenser containments (ice condensers use WCOBRA/TRAC which is an updated NRC-approved methodology)

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- The TR would demonstrate that the methodology used in WCAP-10325-P-A is very conservative (and this was shown on a slide comparing post-accident containment pressure curves for WCAP-10325-P-A vs. WCOBRA/TRAC) – note that this comparison demonstrated there is significant margin and no safety concern
- A pre-submittal meeting for the TR would be scheduled for Q4 CY2017 and the TR would be submitted Q1 CY2018
- Licensees could incorporate the TR into their licensing basis via 10 CFR 50.59, following NRC approval of the TR thus eliminating the need for licensees to submit license amendment requests (LARs) and for NRC to review them
- Licensee would also incorporate WCAP-17221 into their licensing basis. WCAP-17221 demonstrates the conservatism in WCAP-10325-P-A
- Licensees would retain the WCAP-10325-P-A methodology as their analysis of record (AOR) – the addition of the reference to the approved TR would demonstrate how conservative the AOR is
- In LARs that have an impact on accident analysis and post-accident containment pressure, where NRC would previously have asked questions on Infogram 14-1, licensees could indicate in those LARs that Infogram 14-1 would be addressed separately from the LAR (this would not be interpreted as a licensing commitment which was the issue at a licensee pre-submittal meeting earlier this year)
- Other Nuclear Safety Advisory Letters (NSALs) which identify errors in WCAP-10325-P-A (i.e., 06-6, 11-5, 14-2) would not be addressed by the TR, therefore licensees would be responsible for addressing these other NSALs in LARs that have an impact on post-accident containment pressure

A list of attendees is available in the Agencywide Documents Access and Management System (ADAMS) as Accession No. ML17128A369. Additional supporting materials may be found in ADAMS as Accession No. ML17081A081.

Project No. 694

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ADAMS Accession Nos.: Pkg.: ML17128A366; Summary: ML17128A368; Attendees List: ML17128A369; *concurrence via e-mail **NRC-001**

OFFICE	NRR/DPR/PLPB/PM	NRR/DSS/SRXB*	NRR/DLR/RERP*	NRR/DPR/PLPB/BC	NRR/DPR/PLPB/PM
NAME	BBenney	EOesterle	BBeasley	DMorey	BBenney
DATE	5/8/17	7/5/17	7/5/17	8/1/17	8/2/17

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