

Vogtle PEmails

From: Kallan, Paul
Sent: Thursday, May 04, 2017 10:02 AM
To: Vogtle PEmails
Subject: Acceptance Letter for Vogtle LAR 17-012, Addition of Steam Generator System (SGS) Thermal Relief Valves (CAC RP9621)1.docx
Attachments: Acceptance Letter for Vogtle LAR 17-012, Addition of Steam Generator System (SGS) Thermal Relief Valves (CAC RP9621)1.docx

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Subject: Acceptance Letter for Vogtle LAR 17-012, Addition of Steam Generator System (SGS) Thermal Relief Valves (CAC RP9621)1.docx
Sent Date: 5/4/2017 10:01:35 AM
Received Date: 5/4/2017 10:01:36 AM
From: Kallan, Paul

Created By: Paul.Kallan@nrc.gov

Recipients:
"Vogtle PEmails" <Vogtle.PEmails@nrc.gov>
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MESSAGE	3	5/4/2017 10:01:36 AM
Acceptance Letter for Vogtle LAR 17-012, Addition of Steam Generator System (SGS) Thermal Relief Valves (CAC RP9621)1.docx	31080	

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Return Notification: No
Reply Requested: No
Sensitivity: Normal
Expiration Date:
Recipients Received:

May 4, 2017

Mr. Brian H. Whitley, Director
Regulatory Affairs
Southern Nuclear Operating Company, Inc.
42 Inverness Center Parkway
BIN B237
Birmingham, AL 35242

SUBJECT: ACCEPTANCE REVIEW OF SOUTHERN NUCLEAR OPERATING COMPANY'S REQUEST FOR LICENSE AMENDMENT (LAR 17-012) AND EXEMPTION FOR THE VOGTLE ELECTRIC GENERATING PLANT UNITS 3 AND 4: ADDITION OF STEAM GENERATOR SYSTEM (SGS) THERMAL RELIEF VALVES (CAC NO. RP9621)

Dear Mr. Whitley:

By letter dated April 21, 2017 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML17111A958, Southern Nuclear Operating Company (SNC) submitted a request for a license amendment and an exemption to Combined License (COL) Numbers NPF-91 and NPF-92, for the Vogtle Electric Generating Plant (VEGP) Units 3 and 4, respectively. The requested amendment proposes to depart from approved AP1000 Design Control Document (DCD) Tier 2 information (text, tables and figures) as incorporated into the Updated Final Safety Analysis Report (UFSAR) as plant-specific DCD information, and also proposes to depart from involved plant-specific Tier 1 information (and associated COL Appendix C information). Specifically, the amendment request proposes changes to COL Appendix C (and plant-specific Tier 1) Table 2.2.4-1 and Figure 2.2.4-1 to add two main feedwater thermal relief valves and two start-up feedwater thermal relief valves. The proposed COL Appendix C (and plant-specific DCD Tier 1) changes require additional changes to corresponding Tier 2 information in UFSAR Chapter 3 and 10.

SNC has also requested an exemption from the provisions of Title 10 of the *Code of Federal Regulations* (10 CFR) Part 52, Appendix D, Section III.B, "Design Certification Rule for the AP1000 Design, Scope and Contents," to allow a departure from the elements of the certification information in Tier 1 of the generic Design Control Document.

The purpose of this letter is to provide the results of the U.S. Nuclear Regulatory Commission (NRC) staff's acceptance review of these requests. The acceptance review was performed to determine if there is sufficient technical information in scope and depth to allow the NRC staff to complete its detailed technical reviews. The acceptance review is also intended to identify whether the application has any readily apparent information insufficiencies in its characterization of the regulatory requirements or the licensing basis of the plant.

Consistent with Section 50.90 of the 10 CFR, an amendment to the license must fully describe the changes requested, and follow as far as applicable, the form prescribed for original applications. Section 52.79 of the 10 CFR addresses content of technical information required. This section stipulates that the submittal address the design and operating characteristics, unusual or novel design features, and principal safety considerations.

The NRC staff has reviewed your application and concluded that it does provide technical information in sufficient detail to enable the NRC staff to complete its detailed technical review and make an independent assessment regarding the acceptability, of the proposed amendment and exemption, in terms of regulatory requirements and the protection of public health and safety and the environment. Given the lesser scope and depth of the acceptance review as compared to the detailed technical review, there may be instances in which issues that impact the NRC staff's ability to complete the detailed technical review are identified despite completion of an adequate acceptance review. You will be advised of any further information needed to support the NRC staff's detailed technical review by separate correspondence.

If you have any questions, please contact me at (301) 415-2809 or Paul.Kallan@nrc.gov.

Sincerely,

/RA/

Paul Kallan, Senior Project Manager
Licensing Branch 4
Division of New Reactor Licensing
Office of New Reactors

Docket No(s): 52-025
52-026