

PMSummerColpEM Resource

From: Kallan, Paul
Sent: Thursday, May 04, 2017 9:06 AM
To: PMSummerColpEM Resource
Subject: Summer LAR 17-07 - Addition of In-Containment Refueling Water Storage Tank Lower Narrow Range Level Instrumentation
Attachments: Acceptance Letter for Summer LAR 17-07, Addition of IRWST Lower Narrow Range Level Instrumentation (CAC RG3041).docx

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Subject: Summer LAR 17-07 - Addition of In-Containment Refueling Water Storage Tank
Lower Narrow Range Level Instrumentation
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Reply Requested: No
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May 4, 2017

Mr. Ronald A. Jones, Vice-President
South Carolina Electric & Gas Company
New Nuclear Deployment
P. O. Box 88
Jenkinsville, SC 29065

SUBJECT: ACCEPTANCE REVIEW OF SOUTH CAROLINA ELECTRIC & GAS COMPANY'S REQUEST FOR LICENSE AMENDMENT (LAR 17-07) AND EXEMPTION FOR THE VIRGIL C. SUMMER NUCLEAR STATION UNITS 2 AND 3 RE: ADDITION OF IN-CONTAINMENT REFUELING WATER STORAGE TANK LOWER NARROW RANGE LEVEL INSTRUMENTATION (CAC NO. RG3041)

Dear Mr. Jones:

By letter dated May 2, 2017 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML17122A353), South Carolina Electric & Gas Company (SCE&G) submitted a request for a license amendment and an exemption to Combined License (COL) Numbers NPF-93 and NPF-94, for the Virgil C. Summer Nuclear Station (VCSNS) Units 2 and 3, respectively. The amendment request proposes changes to the Protection and Safety Monitoring System (PMS) including the reactor trip system (RTS) and the engineered safety feature actuation system (ESFAS), the passive core cooling system (PXS), the steam generator blowdown system (BDS), and the spent fuel pool cooling system (SFS). In addition, revisions are proposed to COL Appendix A, Technical Specifications.

SCE&G has also requested an exemption from the provisions of Title 10 of the *Code of Federal Regulations* (10 CFR) Part 52, Appendix D, Section III.B, "Design Certification Rule for the AP1000 Design, Scope and Contents," to allow a departure from the elements of the certification information in Tier 1 of the generic Design Control Document.

The purpose of this letter is to provide the results of the U.S. Nuclear Regulatory Commission (NRC) staff's acceptance review of these requests. The acceptance review was performed to determine if there is sufficient technical information in scope and depth to allow the NRC staff to complete its detailed technical reviews. The acceptance review is also intended to identify whether the application has any readily apparent information insufficiencies in its characterization of the regulatory requirements or the licensing basis of the plant.

Consistent with Section 50.90 of the 10 CFR, an amendment to the license must fully describe the changes requested, and follow as far as applicable, the form prescribed for original applications. Section 52.79 of the 10 CFR addresses content of technical information required. This section stipulates that the submittal address the design and operating characteristics, unusual or novel design features, and principal safety considerations.

The NRC staff has reviewed your application and concluded that it does provide technical information in sufficient detail to enable the NRC staff to complete its detailed technical review and make an independent assessment regarding the acceptability, of the proposed amendment and exemption, in terms of regulatory requirements and the protection of public health and safety and the environment. Given the lesser scope and depth of the acceptance review as compared to the detailed technical review, there may be instances in which issues that impact the NRC staff's ability to complete the detailed technical review are identified despite completion of an adequate acceptance review. You will be advised of any further information needed to support the NRC staff's detailed technical review by separate correspondence.

If you have any questions, please contact me at (301) 415-2809 or Paul.Kallan@nrc.gov.

Sincerely,

/RA/

Paul Kallan, Senior Project Manager
Licensing Branch 4
Division of New Reactor Licensing
Office of New Reactors

Docket No(s): 52-027
52-028