

FEB 2 1973

Docket Nos. 50-237, 50-249,
50-254 and 50-265

Commonwealth Edison Company
ATTN: Mr. Byron Lee, Jr.
Assistant to the President
Post Office Box 767
Chicago, Illinois 60690

Gentlemen:

The control rod drop accident is one of the design basis accidents analyzed in the Final Safety Analysis Reports (FSARs) for your reactors. The assumptions and parameters used in the analysis of the consequences of the control rod drop accident have been under review within the Directorate of Licensing and by the General Electric Company. The General Electric Company has issued a topical report NEDO-10527, "Rod Drop Accident Analysis for Large Boiling Water Reactors", dated March 1972 that is applicable to reactor cores with temporary poison curtains. In addition, the General Electric Company has issued NEDO-10527, Supplement 1, "Rod Drop Accident Analysis for Large Boiling Water Reactors, Addendum 1, Multiple Enrichment Cores with Axial Gadolinium". Quad-Cities 1 and 2 and Dresden 2 currently are loaded with axially distributed gadolinia-bearing fuel but in a different configuration than that used as a basis for the analysis provided in NEDO-10527, Supplement 1. Therefore, it is not apparent that either NEDO-10527 or Supplement 1 thereto is fully applicable to Quad-Cities 1 and 2 and Dresden 2.

It is requested that you submit, within 60 days from the date of this letter, an analysis which supports the applicability of either NEDO-10527 or its Supplement 1 to the control rod drop accident analysis for Quad-Cities 1 and 2 and Dresden 2 or to provide an analysis which is applicable to these reactors. It is also requested that a discussion with supporting data be provided for Dresden 2 and 3 and Quad-Cities 1 and 2 concerning the expected maximum reactivity worth of insequence control rods and of the maximum reactivity worth of an out-of-sequence rod assuming a single operator error. This information will be used in conjunction with the rod drop accident analysis to evaluate current technical specifications relative to control rods.

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If your submittal is in the form of a single document involving all four units, please submit one signed original and fifty-nine additional copies. If your submittal is in the form of a separate document covering each station, each document should be submitted with one signed original and thirty-nine additional copies.

If you are planning changes to your reactor fuel design in the next one or two refueling outages, it would be mutually beneficial for us to be informed of your plans as early as is practical.

Sincerely,

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Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Directorate of Licensing

cc: Mr. Charles Whitmore
President and Chairman
Iowa-Illinois Gas and
Electric Company
206 East Second Avenue
Davenport, Iowa 52801

John W. Rowe, Esquire
Isham, Lincoln & Beale
Counselors at Law
One First National Plaza
Chicago, Illinois 60670

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604 Liberty Street
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