

Commonwealth Edison Company

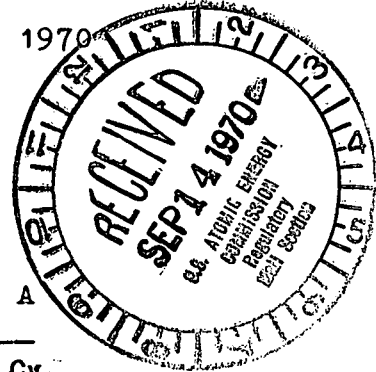
ONE FIRST NATIONAL PLAZA ★ CHICAGO, ILLINOIS

Address Reply to:

POST OFFICE BOX 767 ★ CHICAGO, ILLINOIS 60690

September 10, 1970

Dr. Peter A. Morris, Director
Division of Reactor Licensing
U.S. Atomic Energy Commission
Washington, D. C. 20545



Subject: Proposed Change No. 5 to Appendix A
of DPR-19, AEC Dkt 50-237.

Dear Dr. Morris:

Regulatory

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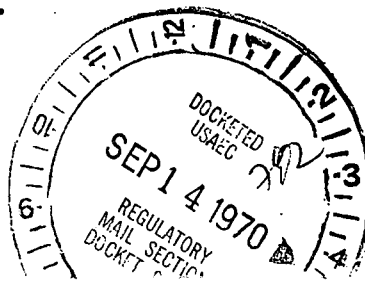
Pursuant to 10CFR50.59 and paragraph 3.B of Facility License DPR-19, Commonwealth Edison hereby submits Proposed Change No. 5 to Appendix A of DPR-19 (Dresden Unit 2). The changes requested in this Proposed Change are in Table 3.2.1 on Instrumentation That Initiates Primary Containment Isolation Functions and Section 4.7 on Containment Systems. The specific changes are as follows:

On page 46, change the High Flow Main Steamline Trip Level Setting to read: " $\leq 140\%$ of rated steam flow."

On page 53, line 6 of the second full paragraph, change "120% of rated steam flow" to "140% of rated steam flow."

On page 131, section 4.7.D.1.b.(2), change "50% of rated" to "100% of rated."

Pursuant to 10CFR50.59 and paragraph 3.B of Facility License DPR-19, an appropriate Safety Analysis Report in support of Proposed Change No. 5 to Appendix A of DPR-19 is attached hereto as Exhibit 1. In our opinion, the Proposed Change does not result in hazards different from or greater than those analyzed in the Final Safety Analysis Report. Specifically, there is (1) no increase in the probability of, or (2) no increase in the possible consequences of, or (3) no creation of a credible probability of an accident or equipment malfunction different from those previously evaluated in the FSAR. Therefore, the margin of safety as defined in the Basis for any Technical Specification is not reduced.



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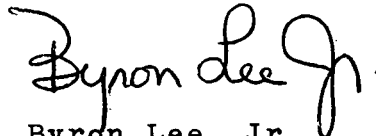
September 10, 1970

We feel this Proposed Change is necessary to prevent inadvertent isolation of the reactor vessel at 100% power. Because we plan to attain 100% power during the power test program on or about September 20, 1970, it would be greatly appreciated if this Proposed Change could be approved at the earliest possible date.

Proposed Change No. 5 has been reviewed and approved by Commonwealth Edison's Nuclear Review Board.

In addition to three signed originals, 19 copies of Proposed Change No. 5 are also submitted.

Very truly yours,



Byron Lee, Jr.
Assistant to the President

SUBSCRIBED and SWORN to
before me this 10th day
of September, 1970.

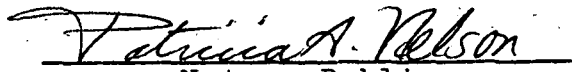

Patricia A. Nelson
Notary Public

EXHIBIT IMain Steamline High Flow Trip Level Setting

Functional testing of the main steam isolation valves (MSIV) is required by Technical Specification 4.7.D.1.b. During closure testing conducted at 75% power, two cases of inadvertent vessel isolation have occurred; this isolation was caused by a steam flow restrictor excessive flow signal. To prevent this inadvertent isolation of the reactor vessel and to improve operational flexibility during MSIV testing, the 120% rated steam flow setting is changed to 140% rated steam flow. Reference for the basis for this change is made to paragraph 3.2, page 53, of the Dresden Unit 2 Technical Specifications and its referenced sections 14.2.3.9 and 14.2.3.10 of the FSAR. The steamline break accident analysis is based on a 200% flow restrictor and its anticipated maximum mass flow and radioactivity release. This proposed change requires no additional analyses. In the improbable event of a break causing less than 140% rated steam flow, the present temperature monitoring system will provide the required isolation.