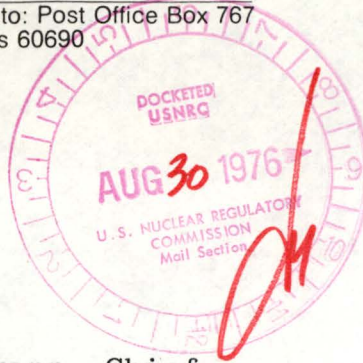




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REGULATORY DOCKET FILE COPY



August 25, 1976

Mr. Dennis L. Ziemann, Chief
Operating Reactors Branch 2
Division of Operating Reactors
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: Mark I Containment Plant Unique Analysis
Amendment 1
Dresden Station Units 2 and 3
Docket No.'s 50-237 and 50-249



Dear Mr. Ziemann:

As was indicated in our letter of August 6, 1976, we are submitting an amendment to the Dresden Station plant unique analysis, Nutech Report COM-01-040, to complete the documentation of the Dresden Unit 3 piping analysis as well as to provide supplemental information related to that report. More specifically, Addendum 1 to the Dresden Unit 2 and 3 plant unique analysis is comprised of four major sections:

1. The results of the stress analysis of piping attached to the Unit 3 torus.
2. The reevaluation of the capacity of the pin connection at the base of the outside columns of Dresden Units 2 and 3. This reevaluation reflects a revision in the calculated capacity of the pin required by the minor modification of the pin cross-section to provide access for torus weighing apparatus. In addition, pin material properties have been reassessed to define the most appropriate value for the ultimate strength of this connection.
3. The results of the torus support column load measuring operation which was recently completed on Dresden Units 2 and 3.
4. The response to NRC Mechanical Engineering Branch Question No. 7 which requested the identification of the design criteria for safety-relief valve piping within the torus.

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In addition, this amendment contains revisions to the main report resulting from additional field surveillance of the Unit 3 piping and from the reevaluation of the outside column pin connection. These changes are provided as Revision 1 to the main body of the plant unique analysis report. All errata identified subsequent to the transmittal of the main report are also identified. The Revision 1 pages are provided in a plastic binder in Section VI of Amendment 1.

The conclusion stated in the Dresden plant unique analysis, Nutech Report COM-01-040, are unchanged by the information contained in Addendum 1; i.e. all torus structural support system components for Units 2 and 3 meet the criteria established for the Mark I Containment Short Term Program as documented in Nutech Report MK1-02-012, Rev. 2. The examination of support system load maldistribution performed as a part of the action plan defined in our letter of February 6, 1976, indicates that the strength ratios (SR) for all support system components do not exceed the criteria of 0.50. In addition, the external piping attached to both Units 2 and 3 has been surveyed and analyzed, and all such piping has been shown to meet ASME Code allowables.

Because all components are within the criteria outlined in Nutech Report MK1-02-012, Rev. 2, it is not necessary to define a plant unique action plan for the modification of the Dresden Units 2 and 3 Mark I Containments. It should also be pointed out that the Dresden plant unique analysis is being reviewed to determine the extent to which the torus-drywell differential pressure (ΔP) of 1.0 psid currently in effect can be reduced within the limits of the Short Term Program criteria. In the event a reduction in ΔP can be justified, we will request your concurrence to operate Dresden Units 2 and 3 at the lower level of ΔP .

One (1) signed original and 39 copies of this letter are submitted for your use. Forty (40) copies of Amendment 1 to the Dresden Units 2 and 3 plant unique analysis (Nutech Report COM-01-040) are being forwarded under separate cover by Nutech.

Very truly yours,



G.A. Abrell
Nuclear Licensing Administrator
Boiling Water Reactors