

ATTACHMENT 1

TECHNICAL SPECIFICATION 3.6.3

CONTAINMENT ISOLATION VALVES

Present Conditions of License

Table 3.6-1 lists power operated valves, check valves and Phase "A" Isolation Valves for the containment isolation.

Included in table 3.6-1 are a Phase "A" Isolation Valve (FCV-682) and the associated check valve for "Instrument Air Supply for H₂ Purge."

Proposed Condition of License

1. Table 3.6-1 would be expanded to include the following valves associated with the Post-LOCA sampling system:

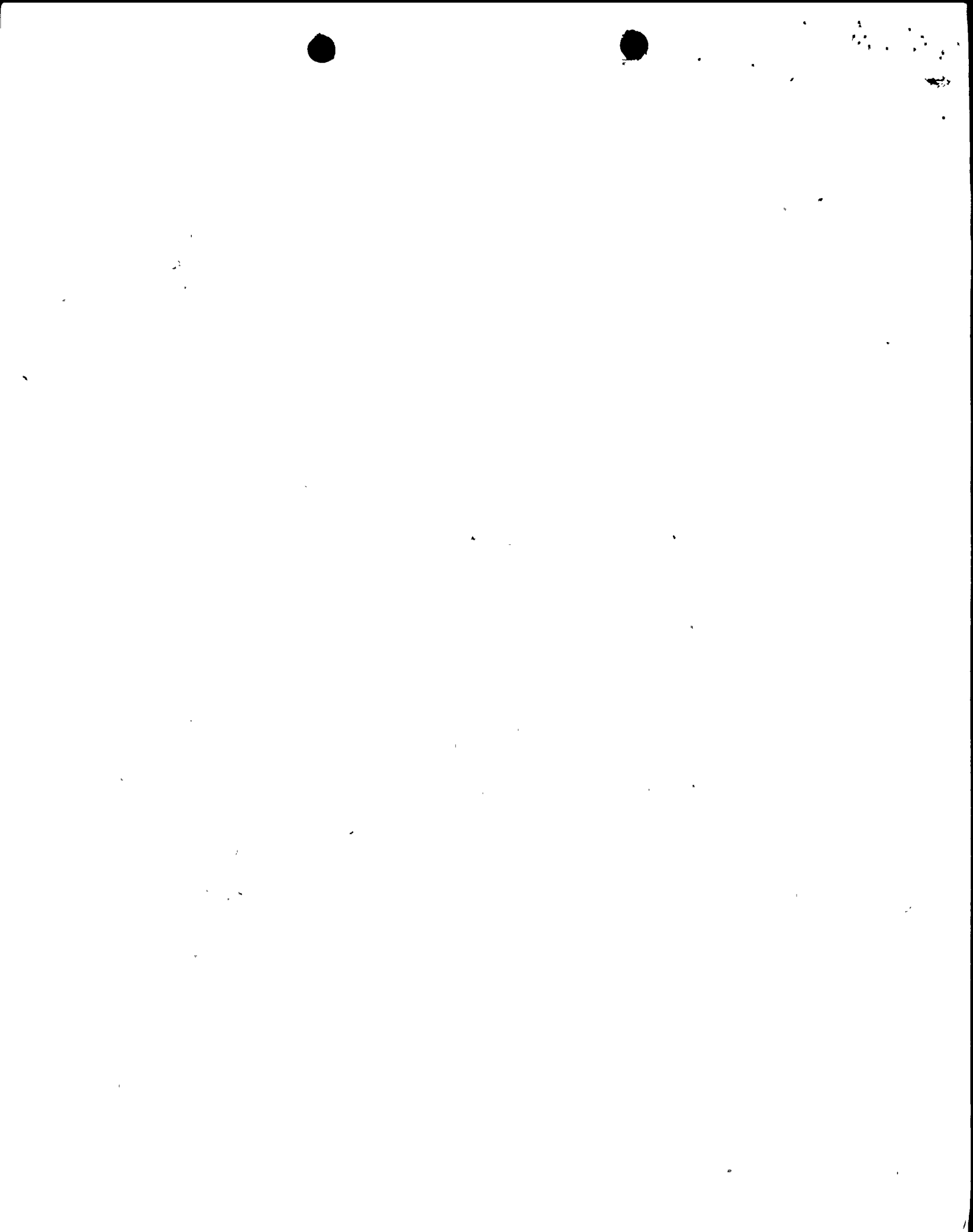
<u>Valve No.</u>	<u>Description</u>
FCV-696	Reactor Cavity Sump Sample Supply IC
FCV-697	Reactor Cavity Sump Sample Supply OC
FCV-698	Containment Air Sample Supply IC
FCV-699	Containment Air Sample Supply OC
FCV-700	Containment Air Sample Return OC
Check valve	Containment Air Sample Return IC

2. Valve FCV-682, and its associated check valve would be deleted from Table 3.6-1.

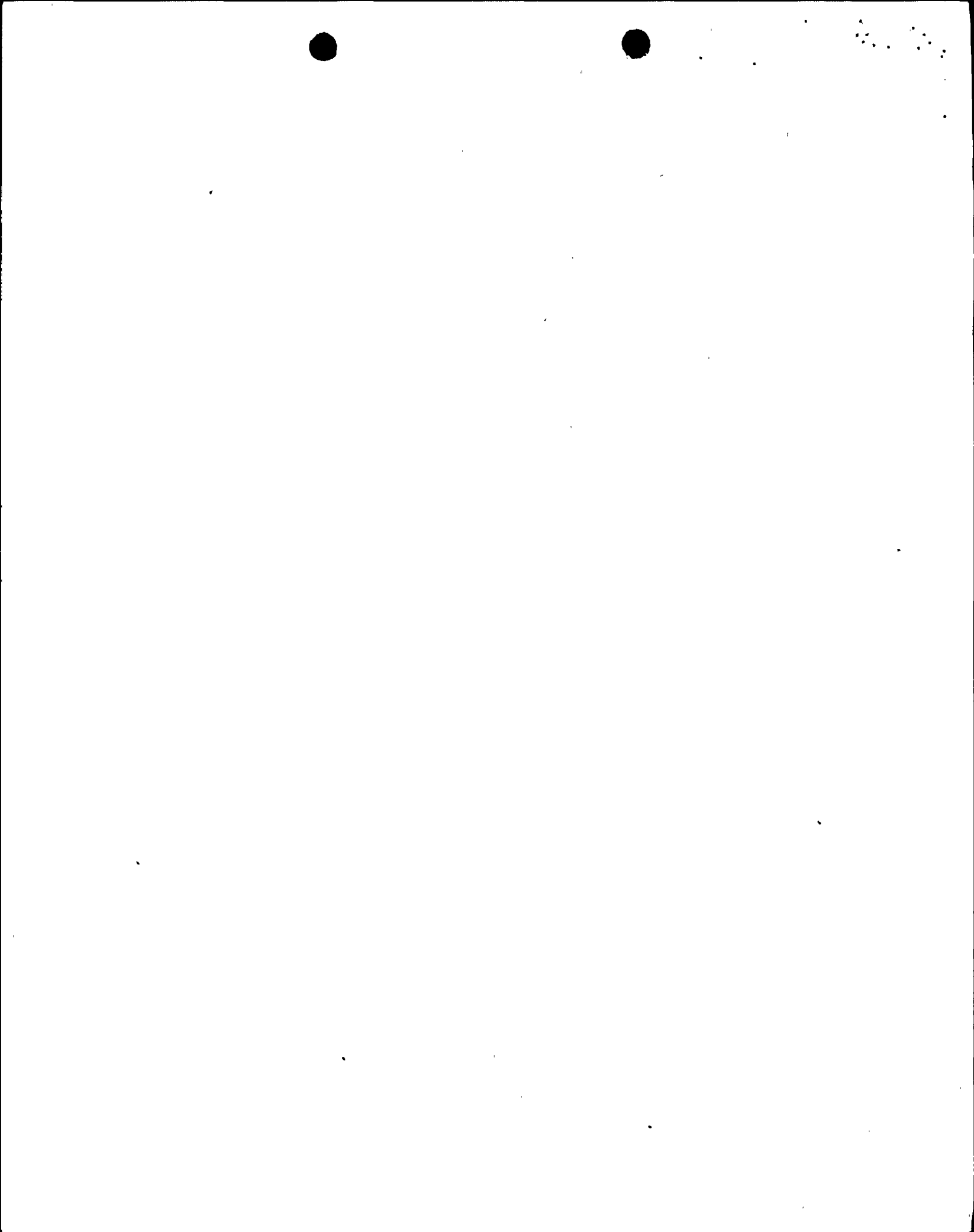
The changes discussed above are shown in the attached changes to Table 3.6-1.

Justification

The installation of the Post Accident Sampling System requires the addition of five power operated containment isolation valves and one check valve to supplement the Post LOCA High Radiation Sampling System. These valves are for taking samples from the reactor cavity sump and the containment air.



These valves are no longer required since compressed air backup for operation of valves in containment has been supplemented by Nitrogen and/or post accident operation is no longer required. The removal of FCV-682 and associated check valve has been reviewed and does not represent an unreviewed safety question as defined in 10 CFR 50.59, as the penetration will be closed by welding caps on the pipe penetration once the valves are removed.



ADDITIONS TO TECHNICAL SPECIFICATION

TABLE 3.6-1

<u>VALVE NO.</u>	<u>FUNCTION</u>	<u>ISOLATION TIME (SEC.)</u>
SECTION E. POWER OPERATED VALVES		
FCV-696	Reactor Cavity Sump Sample (Post LOCA) Supply IC	NA
FCV-697	Reactor Cavity Sump Sample (Post LOCA) Supply OC	NA
FCV-698	Containment Air Sample (Post LOCA) Supply IC	NA
FCV-699	Containment Air Sample (Post LOCA) Supply OC	NA
FCV-700	Containment Air Sample (Post LOCA) Return OC	NA
SECTION F. CHECK VALVES		
Check Valve	Containment Air Sample (Post LOCA) Return IC	NA

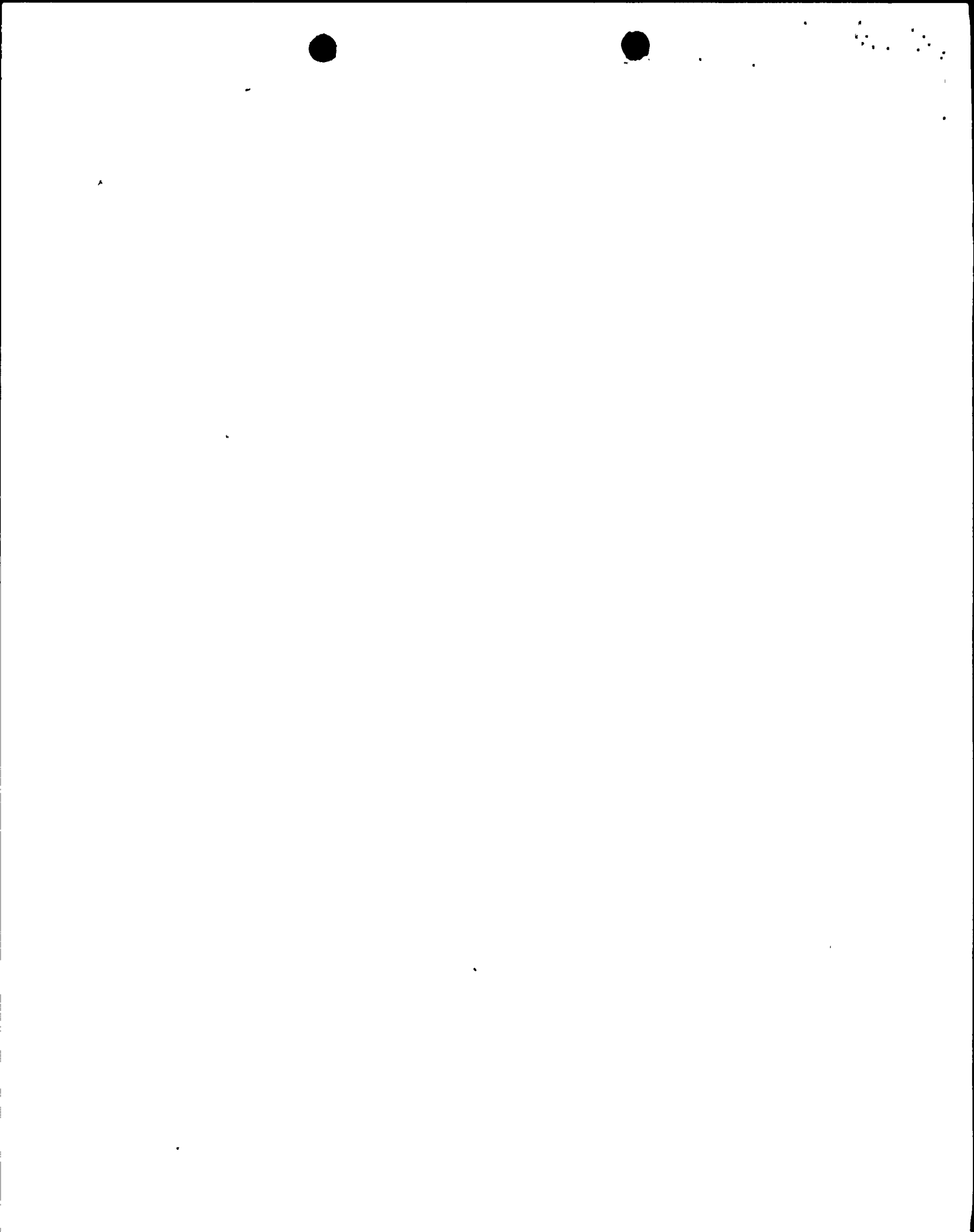


TABLE 3.6-1 (Continued)

DIABLO CANYON - UNIT 1
3/4 6-1B

<u>VALVE NO.</u>	<u>FUNCTION</u>	<u>ISOLATION TIME (Seconds)</u>
FCV-656	Incore Cooler Chilled H ₂ O Return OC	≤ 10
FCV-657	Incore Cooler Chilled H ₂ O Return IC	≤ 10
FCV-682	Instrument Air Supply for H₂ Purge Isol. OC	≤ 10
8029	Primary H ₂ O Supply to Pressurizer Relief Tk. OC	≤ 10
8034A	Pressurizer Relief Tk. to GA IC	≤ 10
8034B	Pressurizer Relief Tk. to GA OC	≤ 10
8045	Pressurizer Relief Tk. N ₂ Supply OC	≤ 10
8149A	Letdown Orifice RO-27 Outlet IC	≤ 10
8149B	Letdown Orifice RO-28 Outlet IC	≤ 10
8149C	Letdown Orifice RO-29 Outlet IC	≤ 10
8152	Letdown Line Isolation OC	≤ 10
8871	ECCS Check Valve Test Line IC	≤ 10
8880	Accumulator N ₂ Fill OC	≤ 10
8883	ECCS Check Valve Test Line OC	≤ 10
8961	ECCS Check Valve Test Line OC	≤ 10
9354A	Pressurizer Steam Space Sample IC	≤ 10
9354B	Pressurizer Steam Space Sample OC	≤ 10
9355A	Pressurizer Liquid Space Sample IC	≤ 10
9355B	Pressurizer Liquid Space Sample OC	≤ 10
9356A	RCS Hot Leg Sample IC	≤ 10
9356B	RCS Hot Leg Sample OC	≤ 10
9357A	Accumulator Sample IC	≤ 10
9357B	Accumulator Sample OC	≤ 10

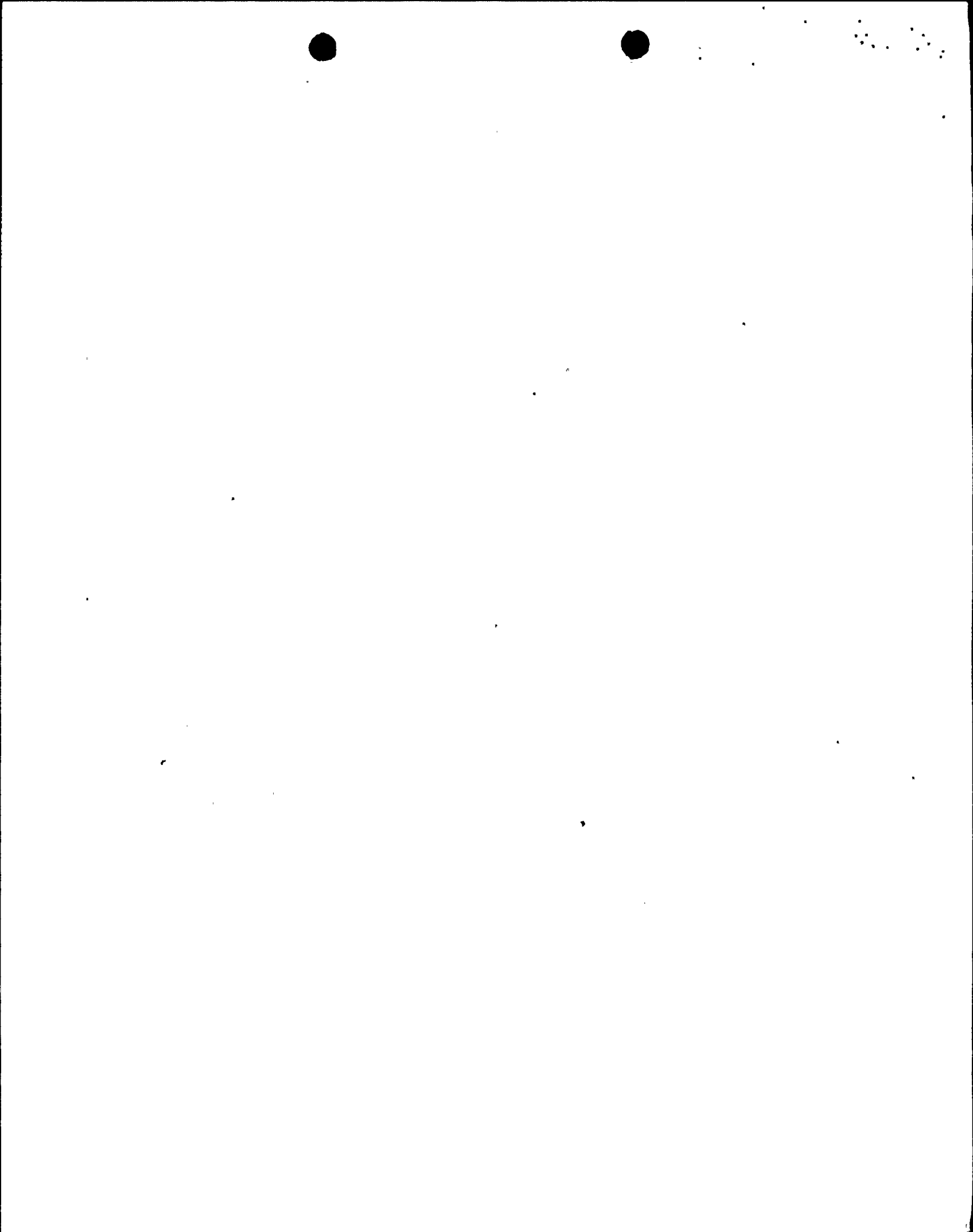


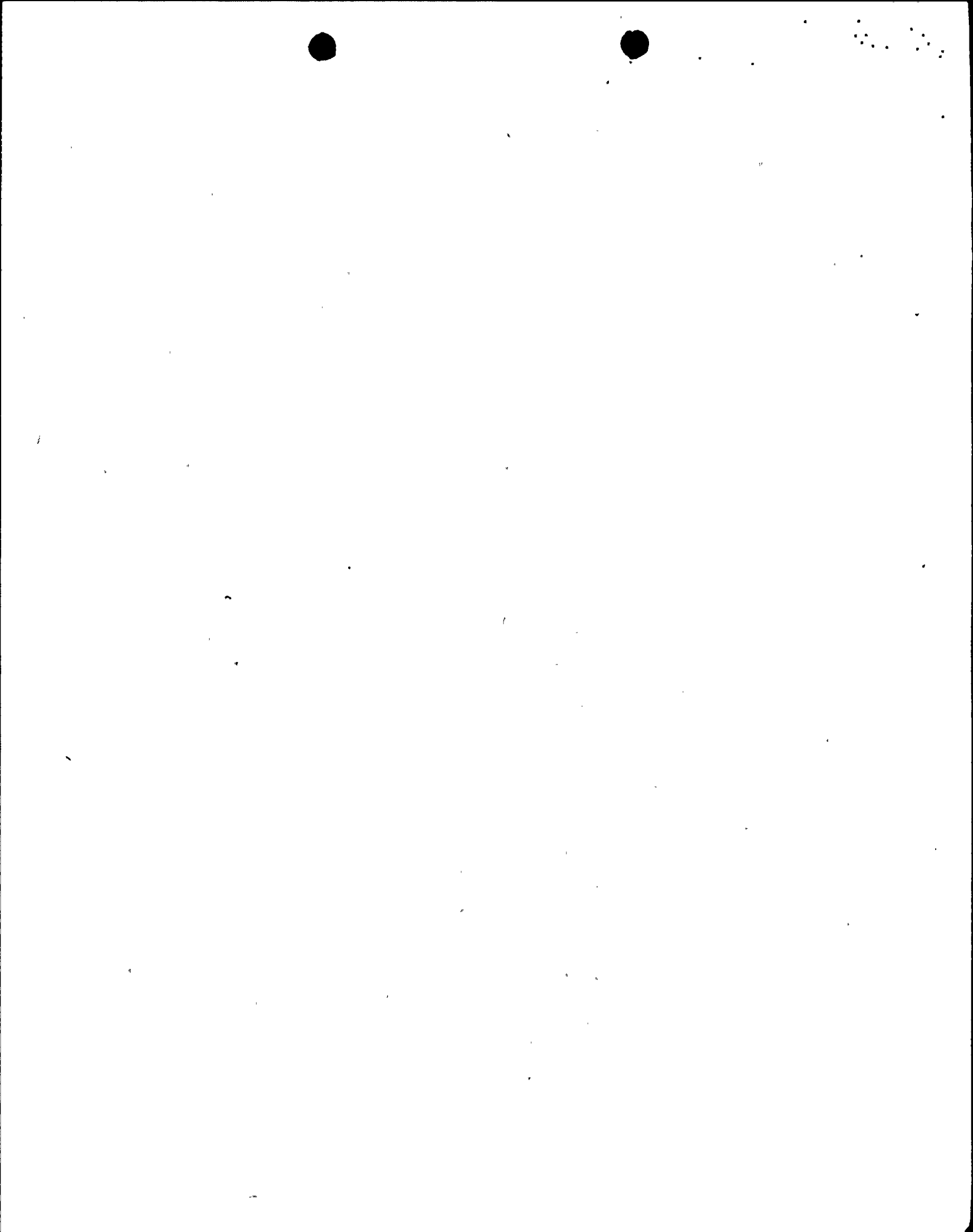
TABLE 3.6-1 (Continued)

<u>VALVE NO.</u>	<u>FUNCTION</u>	<u>ISOLATION TIME (Seconds)</u>
8047	Nitrogen to Pressurizer Relief Tank IC	NA
8109	Seal Water Return IC	NA
8368A thru 8368D	Seal Water to Reactor Coolant Pumps IC	NA
8916	Nitrogen Supply to Accumulators IC	NA
9011A	Containment Spray IC	NA
9011B	Containment Spray IC	NA
Check Valve#	CCW Supply to RCP IC	NA
Check Valve#	CCW Return from RCP (FCV-749 Bypass) IC	NA
Check Valve	CCW Return from RCP (FCV-750 Bypass) IC	NA
Check Valve#	Nitrogen Supply to Stm. Gen. IC	NA
Check Valve#	CCW Supply to Excess Letdown Heat Exchanger OC	NA
Check Valve	Containment Hydrogen Purge Supply IC	NA
Check Valve	Containment Hydrogen Purge Supply IC	NA
Check Valve	Instrument Air for Hydrogen Purge IC	NA
Check Valve	Nitrogen Supply to Reactor Coolant Drain Tank IC	NA
Check Valve	Instrument Air Supply IC	NA
Check Valve	Service Air Supply IC	NA
Check Valve	Containment Air Sample Return IC	NA
Check Valve	Auxiliary Stm. Supply to Containment IC	NA
Check Valve	Containment Fire Water IC	NA
Check Valve	Containment H ₂ Sample Return IC	NA
Check Valve	Containment H ₂ Sample Return IC	NA

*May be opened on an intermittent basis under administrative control (Normally closed manual valves only)

#Not subject to Type C leakage tests

##The provisions of Specification 3.0.4 are not applicable.



ATTACHMENT 2

TECHNICAL SPECIFICATION 6.0

ADMINISTRATIVE CONTROLS

Present Conditions of License

The facility organization for DCPD shows (1) the Senior Vice President, Facilities Development, reporting to the President and Chief Executive Officer, (2) the Shift Foreman reporting directly to the Supervisor of Operations and (3) the Onsite Safety Review Group makes recommendations to the Manager, Nuclear Projects.

Proposed Conditions of License

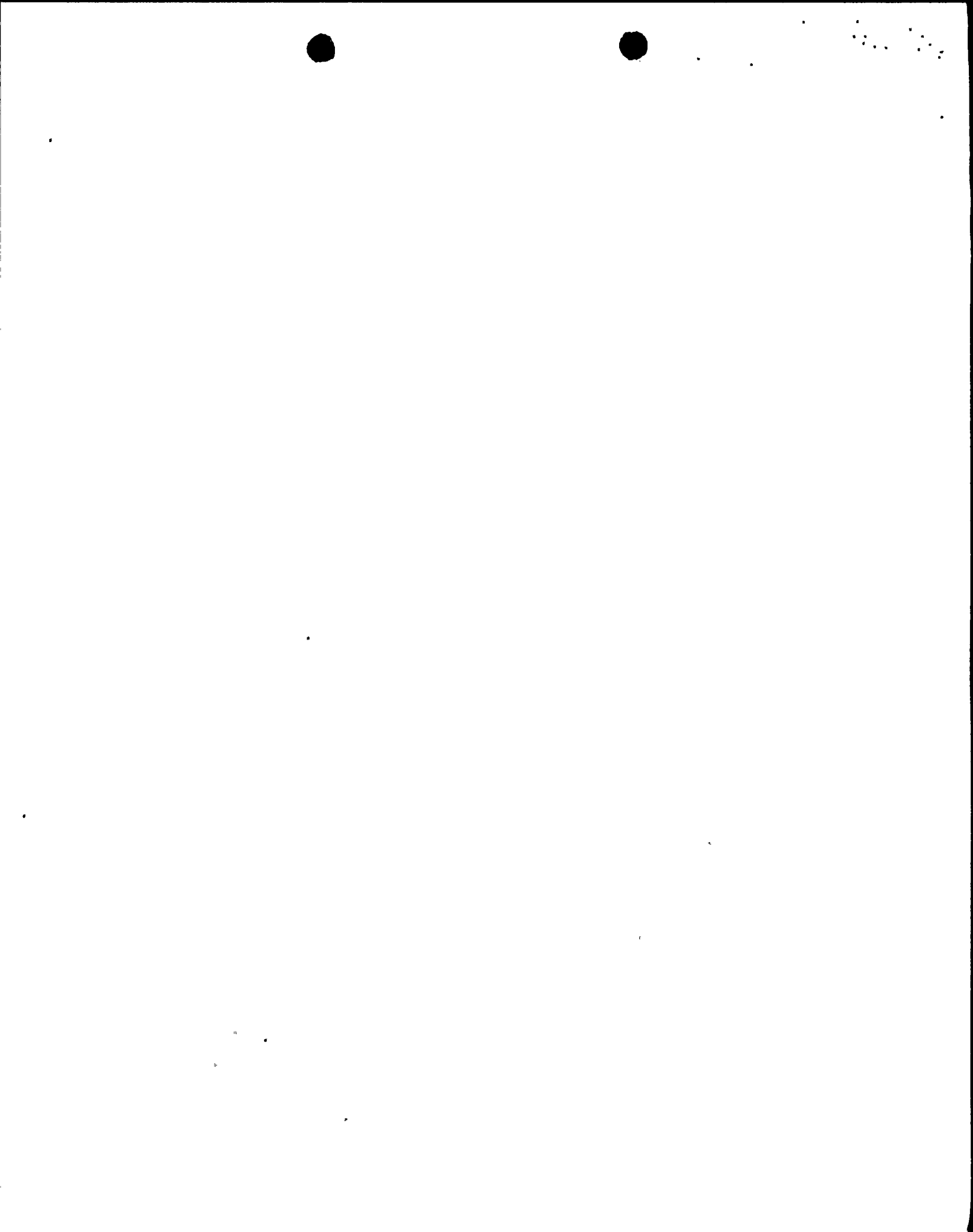
The Senior Vice President, Facilities Development, reports to the President and Chief Operating Officer. The General Operating Foreman reports directly to the Supervisor of Operations and the position of Manager, Nuclear Projects is deleted from the Technical Specifications. The Technical Assistant to the Vice President Nuclear Power Generation assumes the responsibility of interfacing with the Onsite Safety Review Group in place of the Manager, Nuclear Projects. The changes discussed above are shown in the attached pages 6-2, 6-3, 6-5 and 6-8 of the Technical Specifications.

Justification

The change from President and Chief Executive Officer to President and Chief Operating Officer is to correct a typographic error. The General Operating Foreman will supplement the Supervisor of Operations in his duties. He will not assume any authority of the Supervisor of Operations nor his responsibilities unless delegated. This change allows the overall facility organization to maintain a better and closer control of the operators and will provide quicker response to operator questions and problems.

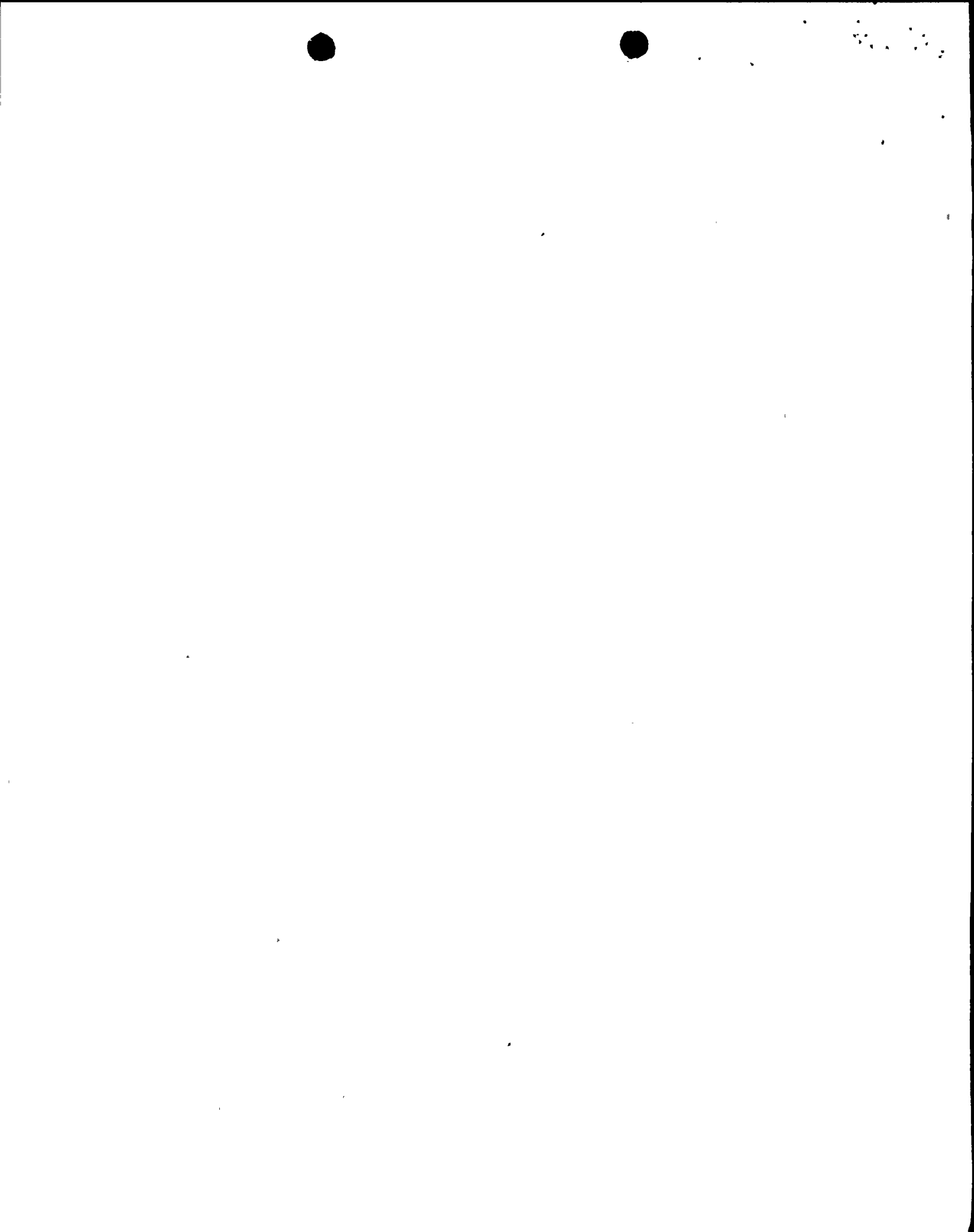
The Technical Specifications focus on the aspects of reactor operation that are important to the health and safety of the public. The position of Manager, Nuclear Projects does not impact on the safe operation of the plant and therefore is not required to be shown in the Technical Specifications. Therefore, the deletion from the Technical Specifications of the position of Manager, Nuclear Projects is considered editorial.

The responsibilities relating to the Onsite Safety Review Group will be transferred to the Technical Assistant to the Vice President, Nuclear Power Generation. In particular, he will review the recommendations of the Onsite Safety Review Group for revised procedures, equipment modifications, maintenance activities, operations activities or other means of improving unit safety.



In the GONPRAC organization the title Project Manager Diablo Canyon will be substituted for Manager, Nuclear Projects.

These changes are editorial and do not involve an unreviewed safety question as defined in 10 CFR 50.59. -



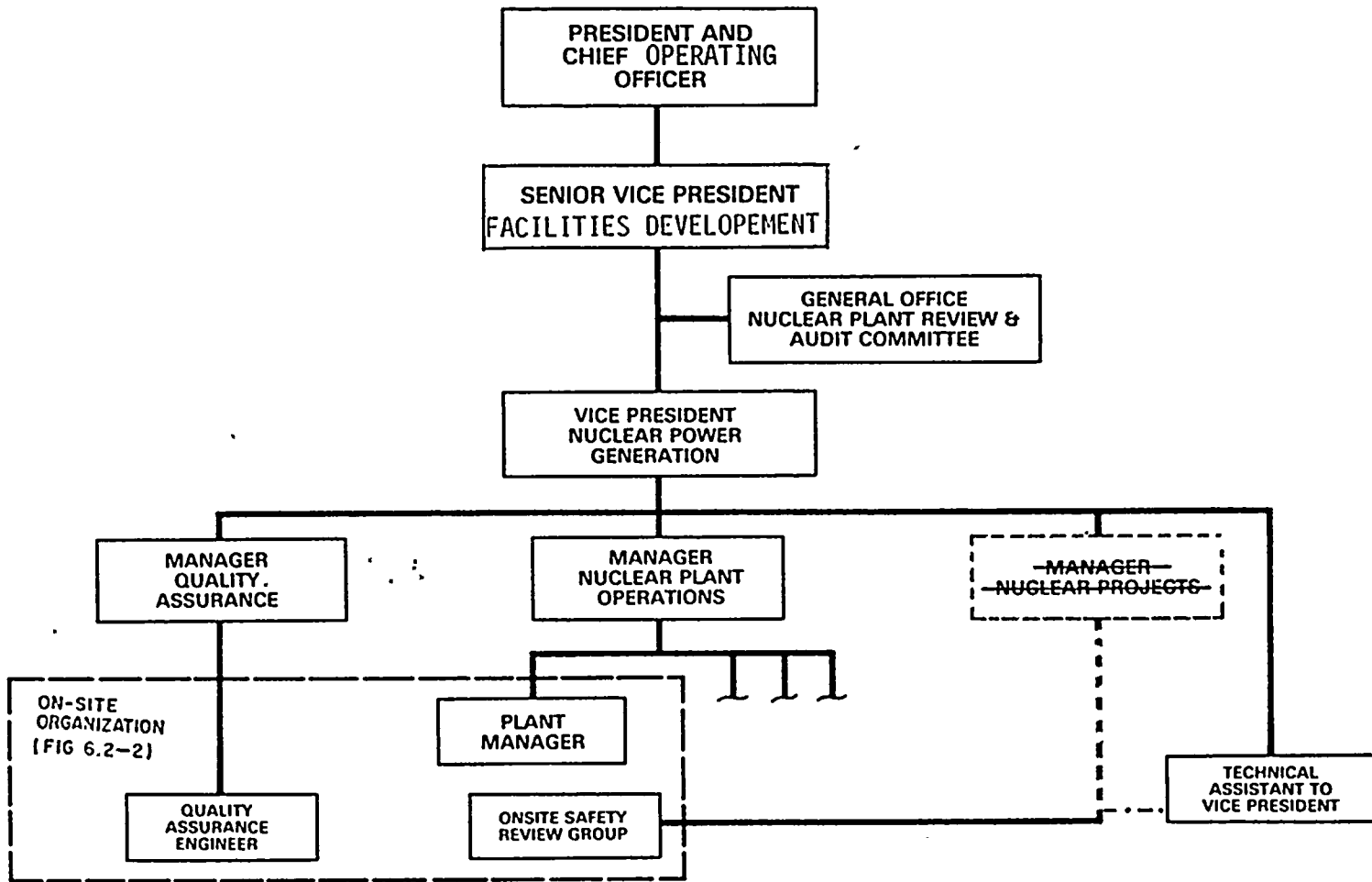
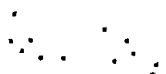


FIGURE 6.2-1

OFFSITE ORGANIZATION FOR FACILITY MANAGEMENT AND TECHNICAL SUPPORT

----- Present Condition of License to be Deleted
-.-.-.- Proposed Condition of License to be Added



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V.P. - NPG

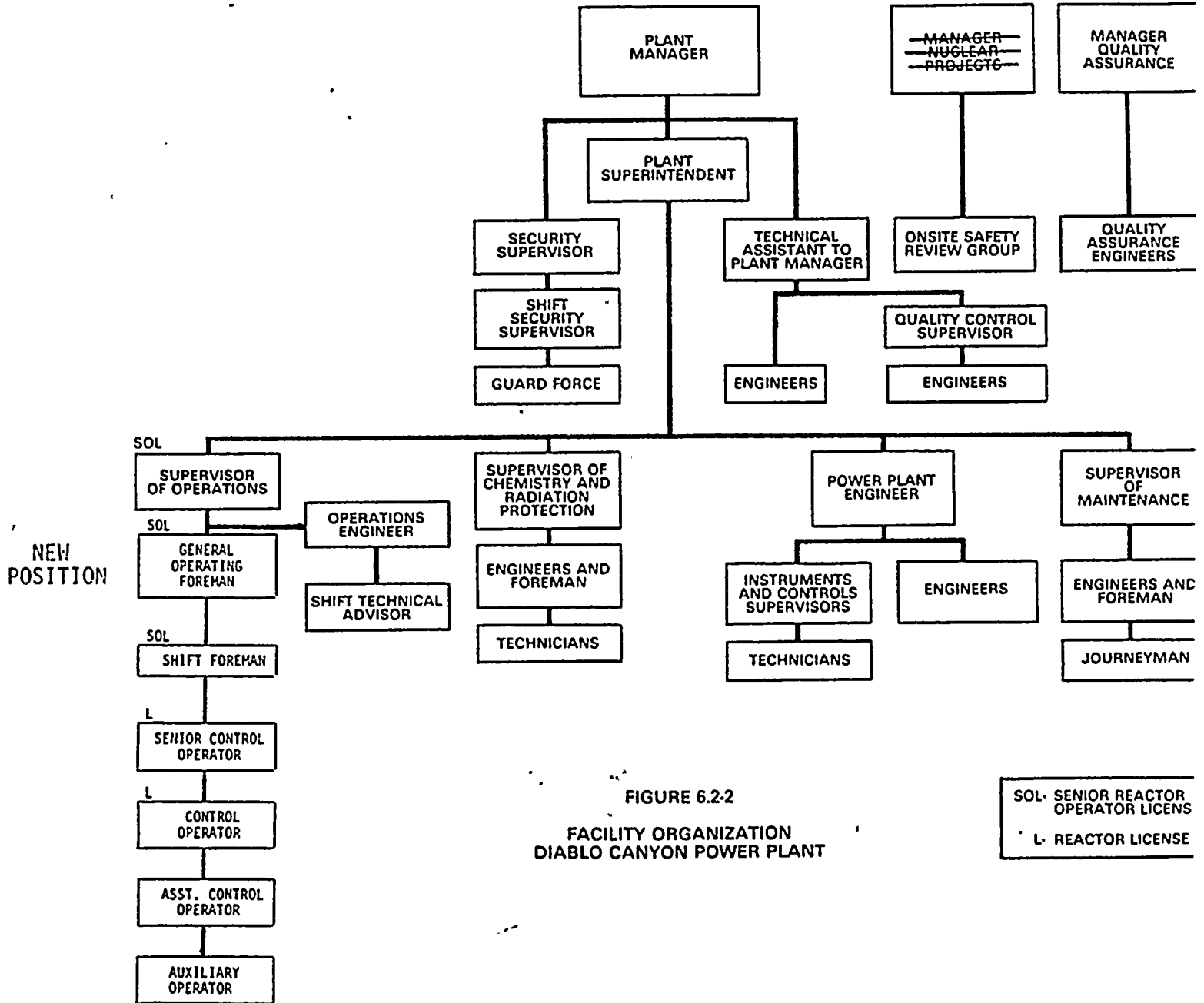
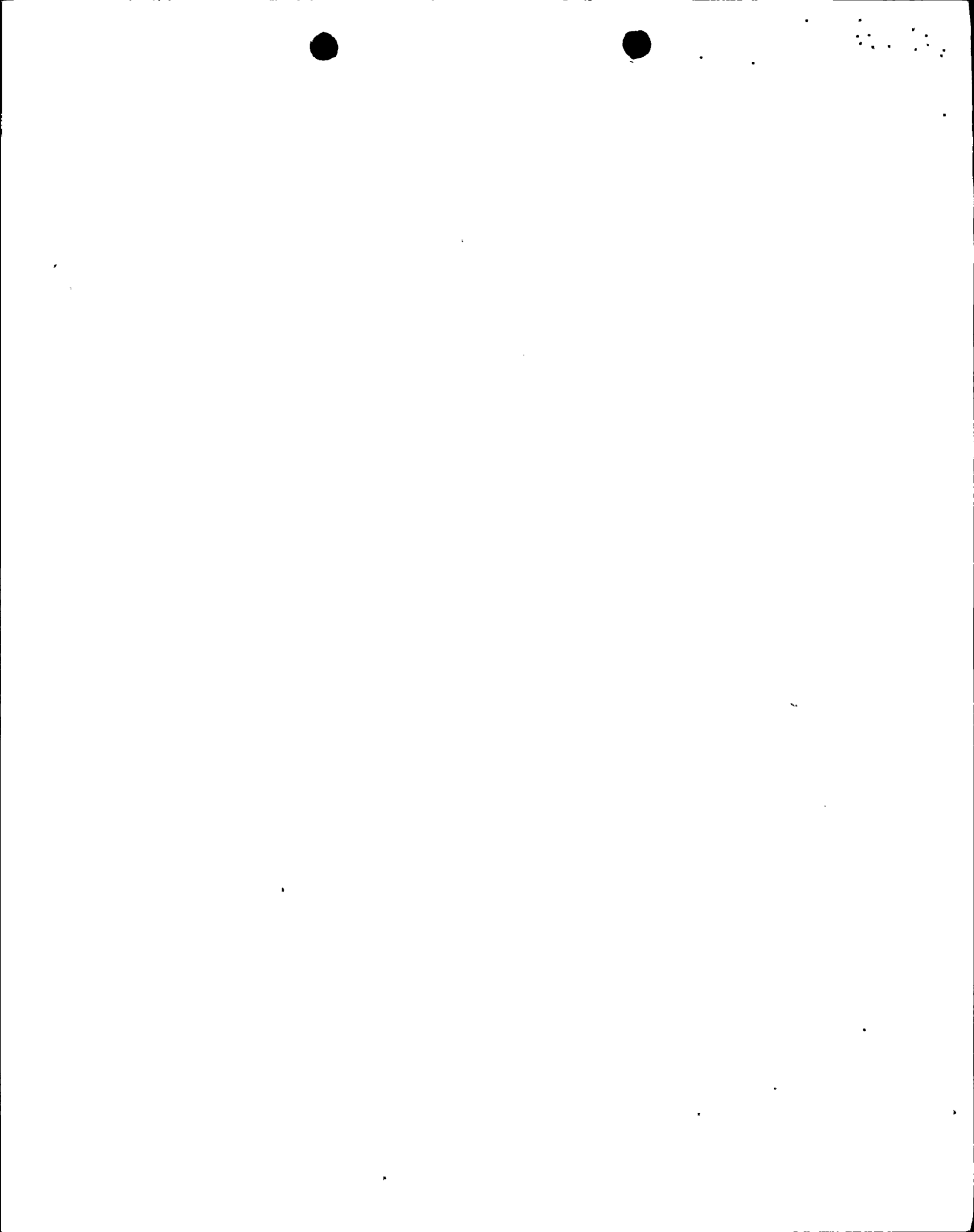


FIGURE 6.2-2
FACILITY ORGANIZATION
DIABLO CANYON POWER PLANT



ADMINISTRATIVE CONTROLS

6.2.3 ONSITE SAFETY REVIEW GROUP (OSRG)

FUNCTION

6.2.3.1 The OSRG shall function to examine unit operating characteristics, NRC issuances, industry advisories, Licensee Event Reports and other sources of plant design and operating experience information, including plants of similar design which may indicate areas for improving plant safety.

COMPOSITION

6.2.3.2 The OSRG shall be composed of at least five engineers located on site.

RESPONSIBILITIES

6.2.3.3 The OSRG shall be responsible for maintaining surveillance of unit activities to provide independent verification* that these activities are performed correctly and that human errors are reduced as much as practical.

AUTHORITY

6.2.3.4 The OSRG shall make detailed recommendations for revised procedures, equipment modifications, maintenance activities, operations activities or other means of improving unit safety to the ~~Manager, Nuclear Projects.~~

6.2.4 SHIFT TECHNICAL ADVISOR

Technical Assistant to Vice President**
Nuclear Power Generation.

6.2.4.1 The Shift Technical Advisor shall provide technical support to the Shift Supervisor in the areas of thermal hydraulics, reactor engineering and plant analysis with regard to the safe operation of the unit.

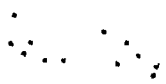
6.3 UNIT STAFF QUALIFICATIONS

6.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions and the supplemental requirements specified in Section A and C of Enclosure 1 of the March 28, 1980 NRC letter to all licensees, except for the Supervisor of Chemistry and Radiation Protection who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the unit staff shall be maintained under the direction of a designated member of the facility staff and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55 and the supplemental requirements specified in Section A and C of Enclosure 1 of the March 28, 1980 NRC letter to all licensees, and shall include familiarization with relevant industry operational experience identified by the OSRG.

*Not responsible for sign-off function.



ADMINISTRATIVE CONTROLS

6.5.2 GENERAL OFFICE NUCLEAR POWER PLANT REVIEW AND AUDIT COMMITTEE (GONPRAC)

FUNCTION

6.5.2.1 The General Office Nuclear Power Plant Review and Audit Committee shall function to provide independent review and audit of designated activities in the areas of:

- a. nuclear power plant operations
- b. nuclear engineering
- c. chemistry and radiochemistry
- d. metallurgy
- e. instrumentation and control
- f. radiological safety
- g. mechanical and electrical engineering
- h. quality assurance practices

COMPOSITION

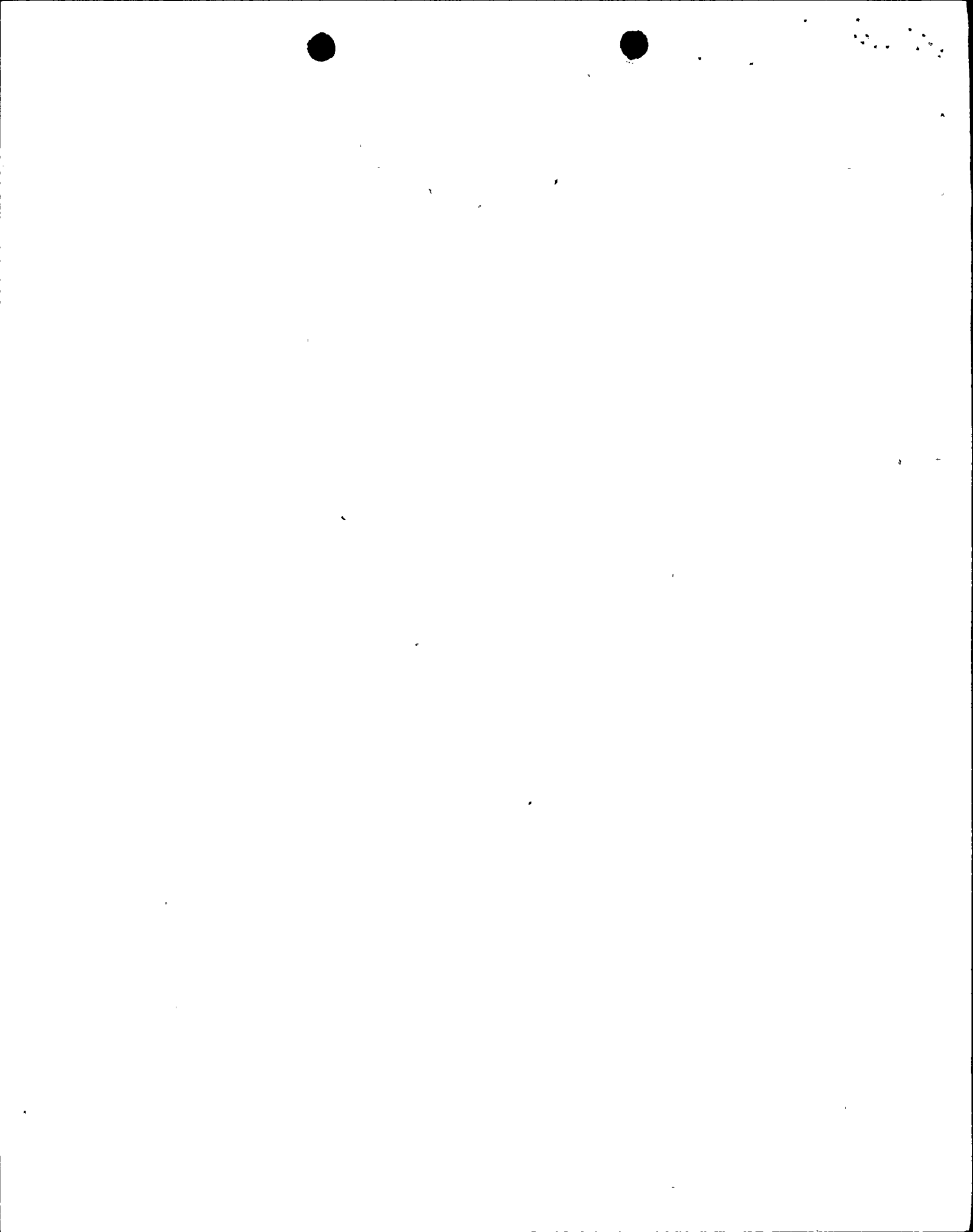
6.5.2.2 The GONPRAC shall be composed of the following:

Chairman:	Vice President, Nuclear Power Generation
Vice Chairman:	Manager, Nuclear Plant Operations*
Member:	Manager, Nuclear Projects*
Member:	Manager, Quality Assurance ← Project Manager, Diablo Canyon *
Member:	Technical Assistant to Vice President, Nuclear Power Generation
Member:	Chief Mechanical and Nuclear Engineer
Member:	Manager, Station Construction

ALTERNATES

6.5.2.3 All alternate members shall be appointed in writing by the GONPRAC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in GONPRAC activities at any one time.

*Members for these positions shall have an academic degree in an engineering or physical science field and a minimum of five years technical experience, of which three years shall be in their respective field of expertise.



ATTACHMENT 3

TECHNICAL SPECIFICATION 3.3.3.8

FIRE DETECTION INSTRUMENTATION

Present Condition of License

In Table 3.3-11, "Fire Detection Instruments," the minimum number of fire detection instruments operable is shown as 1 of 1, 4 of 5, 3 of 4, etc.

Proposed Condition of License

Table 3.3-11, "Fire Detection Instruments," is to be changed reflecting only the minimum required number of instruments at each location; the total number is to be deleted.

The changes discussed above are shown in the attached amended Table 3.3-11.

Justification

There is no logic or redundancy requirements for the number of fire instruments to be operable at each location, only a minimum number requirement.

This change will clarify the actual requirement and does not change any technical criteria. It is primarily an editorial change and will not affect the safety of the various locations. It does not involve an unreviewed safety question as defined in 10 CFR 50.59.

TABLE 3.3-11
FIRE DETECTION INSTRUMENTS

PANEL A

<u>ZONE</u>	<u>INSTRUMENT LOCATION</u>	<u>MINIMUM INSTRUMENTS OPERABLE</u>	
		<u>SMOKE</u>	<u>HEAT OR FLAME</u>
1	Cable Spreading Room - Unit 1	10 of 15	4 of 6
3	4KV Switchgear Bus F Room	1 of 1	N/A
	4KV Switchgear Bus G Room	1 of 1	N/A
	4KV Switchgear Bus H Room	1 of 1	N/A
	4KV Switchgear Ventilation Fan Room	1 of 3	N/A
4	4KV Switchgear Bus F Cable Spreading Area	1 of 1	N/A
	4KV Switchgear Bus G Cable Spreading Area	1 of 1	N/A
	4KV Switchgear Bus H Cable Spreading Area	1 of 1	N/A
7	Containment Electrical Penetration Zone 7	4 of 5	N/A
8	Containment Electrical Penetration Zone 8	3 of 4	N/A
9	Rod Control Programmer/Reactor Trip Breaker Area	3 of 5	N/A
	Battery No. 11 Charger Room	1 of 1	N/A
	Battery No. 12 Charger Room	1 of 1	N/A
	Battery No. 13 Charger Room	1 of 1	N/A
10	480 Volt Bus 1F Area	1 of 1	N/A
	480 Volt Bus 1G Area	1 of 1	N/A
	480 Volt Bus 1H Area	1 of 1	N/A
	Hot Shutdown Panel	1 of 2	N/A
12	Inside Containment	17 of 25 ²	N/A
13	Control Room Ventilation Return/Exhaust	1 of 1	N/A
14	Control Room Ventilation Return/Exhaust	1 of 1	N/A
15	Outside the Auxiliary Salt Water Pump Room	1 of 2	N/A

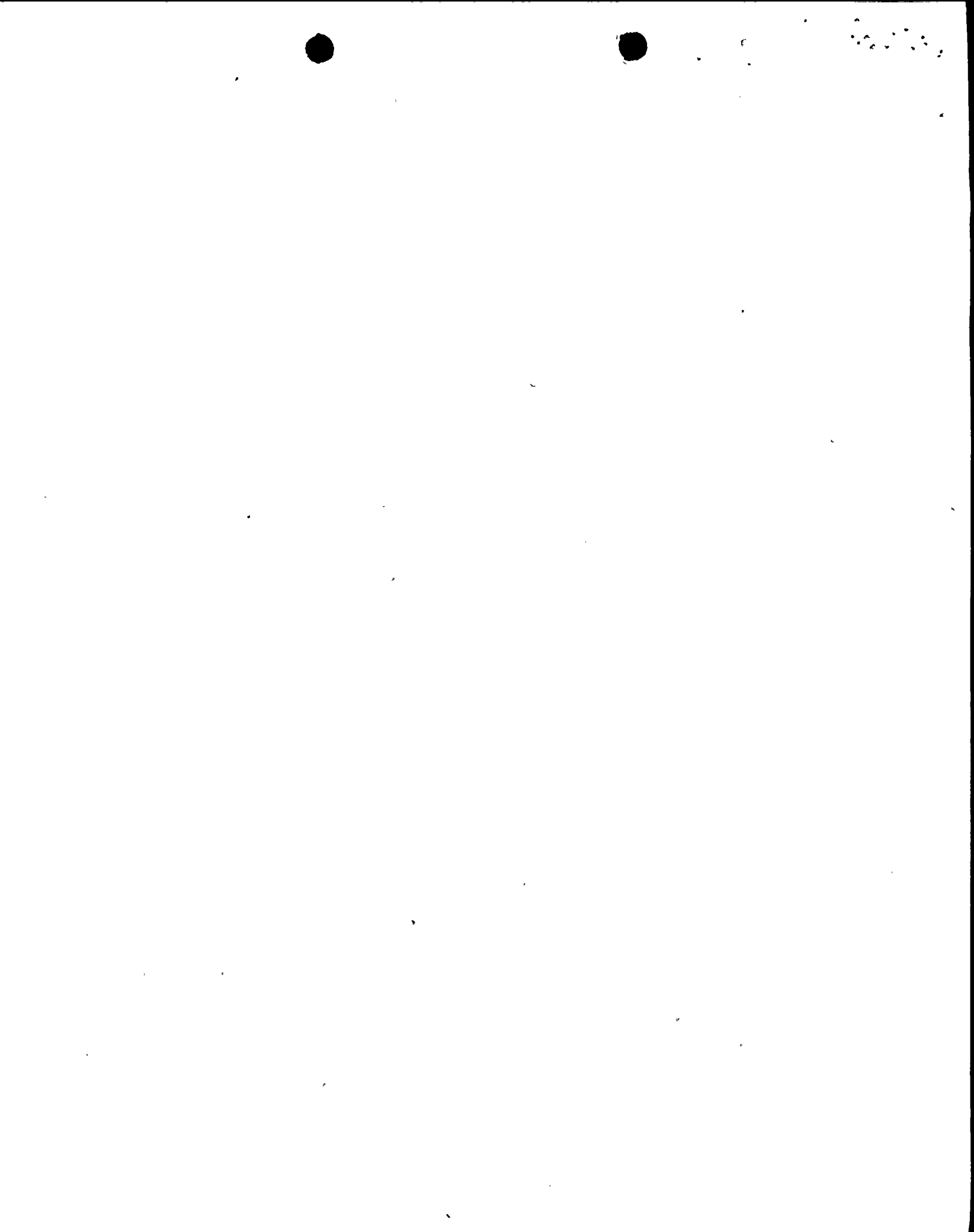


TABLE 3.3-11 (Continued)
FIRE DETECTION INSTRUMENTS

PANEL B

<u>ZONE</u>	<u>INSTRUMENT LOCATION</u>	<u>MINIMUM INSTRUMENTS OPERAE</u>	
		<u>SMOKE</u>	<u>HEAT OR FLA</u>
1	Residual Heat Removal Pump No. 11 Room	1 of 1	N/A
	Residual Heat Removal Pump No. 12 Room	1 of 1	N/A
2	Component Cooling Water Pump No. 11 Room	1 of 1	N/A
	Component Cooling Water Pump No. 12 Room	1 of 1	N/A
	Component Cooling Water Pump No. 13 Room	1 of 1	N/A
	Charging Pump No. 11 Area	1 of 1	N/A
	Charging Pump No. 12 Area	1 of 1	N/A
	Charging Pump No. 13 Area	1 of 1	N/A
	Containment Spray Pump No. 11 Area	1 of 1	N/A
	Containment Spray Pump No. 12 Area	1 of 1	N/A
3	Safety Injection Pump No. 11 Room	1 of 1	N/A
	Safety Injection Pump No. 12 Room	1 of 1	N/A
5	Auxiliary Feedwater Pump No. 1 Area	1 of 1	N/A
	Auxiliary Feedwater Pumps Nos. 2 & 3 Area	1 of 1	N/A
	Boric Acid Transfer Pumps Area	1 of 2	N/A
6	Fire Pumps Area	1 of 1	N/A
7&8	Auxiliary Building Ventilation System Charcoal Filter Bank, EFC-1	12 of 18	6 of 9
11	Fuel Handling Building Ventilation System Charcoal Filter Bank, EFC-5	6 of 8	3 of 4
12	Fuel Handling Building Ventilation System Charcoal Filter Bank, EFC-6	6 of 8	3 of 4
13	Control Room - Control Console	3 of 3	N/A
	Control Room Board	10 of 13	N/A
15	Control Room - Radiation Monitoring	2 of 3	N/A
	Control Room Nuclear Instrumentation	3 of 4	N/A
16(1)	Auxiliary Building Supply Fan Room	1 of 2	N/A
	Control Room Ventilation Equipment Room	1 of 3	N/A
16(2)	Boric Acid Tanks Area	³ 1 of 12	N/A

TABLE 3.3-11 (Continued)
FIRE DETECTION INSTRUMENTS

PANEL B

<u>ZONE</u>	<u>INSTRUMENT LOCATION</u>	<u>MINIMUM INSTRUMENTS OPERABLE</u>	
		<u>SMOKE</u>	<u>HEAT OR FLAME</u>
Not Assigned to Zone	Diesel Generator No. 11 Room	N/A	2 ¹ of 3 ¹
	Diesel Generator No. 12 Room	N/A	2 ¹ of 3 ¹
	Diesel Generator No. 13 Room	N/A	2 ¹ of 3 ¹
Not Assigned to Zone	Solid State Protection System Room	3 ⁴ of 4 ⁴	N/A

¹Heat sensors actuate CO₂ flooding and are tested per Specification 4.7.9.3.c. Specifications 4.3.3.8.1 and 4.3.3.8.2 do not apply.

²The fire detection instruments located within the Containment are not required to be OPERABLE during the performance of Type A Containment Leakage Rate Tests.

³Unit 1 Boric Acid Tank Detectors in Zone 16, Unit 2.

⁴Smoke sensors actuate Halon flooding and are tested per Specification 4.7.9.4.b. Specifications 4.3.3.8.1 and 4.3.3.8.2 do not apply.

