

CATEGORY 1

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SUBJECT: PNO-IV-97-059: on 971024, Unit 2 experienced reactor trip & safety injection from 100% power after main steam isolation valve closed. All plant safety sys operated as designed during event. State of CA notified of event.

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October 27, 1997

PRELIMINARY NOTIFICATION OF EVENT OR UNUSUAL OCCURRENCE PNO-IV-97-059

This preliminary notification constitutes EARLY notice of events of POSSIBLE safety or public interest significance. The information is as initially received without verification or evaluation, and is basically all that is known by Region IV staff in Arlington, Texas on this date.

<u>Facility</u>	<u>Licensee Emergency Classification</u>
Pacific Gas & Electric Co.	X Notification of Unusual Event
Diablo Canyon 2	Alert
Avila Beach, California	Site Area Emergency
Dockets: 50-323	General Emergency
	Not Applicable

Subject: REACTOR TRIP AND NOTIFICATION OF UNUSUAL EVENT

On October 24, 1997, at approximately 8:14 a.m. (PDT), Diablo Canyon Unit 2 experienced a reactor trip and safety injection from 100 percent power after a main steam isolation valve closed. As a result of the safety injection, the licensee declared a Notice of Unusual Event at 8:32 a.m. The Notice of Unusual Event was terminated at 9:38 a.m., after the licensee secured safety injection and determined that plant conditions were stable. All plant safety systems operated as designed during the event.

The main steam isolation valve associated with Steam Generator 4 closed after plant personnel, who were removing scaffolding in the area of the valve, inadvertently stepped on and actuated a close position switch for the valve. Approximately 1.7 seconds after the main steam isolation valve closed, a safety injection signal was generated which initiated a reactor trip and safety injection. The generation of the safety injection signal was an expected response as a result of a rapid steam pressure decrease (21 psig within 100 msec). This was due to the three unisolated steam generators trying to provide steam to the main turbine, which was loaded at 100 percent power.

Plant personnel entered and transitioned through the emergency operating procedures. Operators were able to meet safety injection termination criteria and secured safety injection (including reinitiation of letdown) 23 minutes after the reactor trip. Unit 2 was stabilized in Mode 3, where the plant remained while the licensee investigated the event, took corrective actions, and performed minor repairs. All emergency core cooling system equipment actuated as designed. The emergency diesel generators started, but did not load as offsite power remained available throughout the event. Containment isolation systems actuated as designed. The atmospheric dump valve for Steam Generator 4 opened to relieve pressure after its associated main steam isolation valve closed; however, steam generator pressure did not increase high enough to open main steam safety valves.

Reactor coolant system pressurizer level reached 92 percent during the event as a result of the safety injection and the expected isolation of reactor coolant system letdown. During the event, reactor coolant system pressure increased rapidly enough to actuate one of the three power-operated relief valves. The rapid pressure increase was a result of the high pressurizer level, in conjunction with the expected isolation of pressurizer spray due to containment isolation of the instrument air system. The power-operated relief valve cycled open seven times in a 5- to 7- minute period. The highest reactor coolant system pressure reached was 2260 psig. The two other power-operated relief valves, set to lift at 2335 psig, did not open, and the pressurizer safety

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valves did not open during the event. The licensee determined that the power-operated relief valve which opened operated properly and did not relieve water; however, after the event it did exhibit signs of leakage and was subsequently isolated by its associated block valve.

The NRC senior resident inspector was on site when the event occurred and observed the licensee's response to the event. On October 25, Region IV and NRR personnel held followup telephone conference calls with the licensee to discuss the event, posttrip evaluations, and startup plans. Region IV dispatched a licensed operator examiner to Diablo Canyon to assist in the NRC's evaluation of the event.

On October 25, at approximately 6:25 p.m., Unit 2 was taken critical. Unit 2 synchronized to the grid at 5:18 a.m. on October 26 and is currently operating at 100 percent power. Diablo Canyon Unit 1 remained operating at 100 percent power throughout this event.

The state of California has been notified of this event.

The licensee issued a press release. There was some local press coverage of the event. Region IV received notification of this occurrence from the senior resident inspector at approximately 8:30 a.m. on October 24, 1997.

Region IV has informed the OEDO, PAO, and NRR.

This information has been discussed with the licensee and is current as of 10 a.m. (PST), October 27, 1997.

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