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July 17, 1996



PG&E Letter DCL-96-154

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

Docket No. 50-275, OL-DPR-80

Docket No. 50-323, OL-DPR-82

Diablo Canyon Units 1 and 2

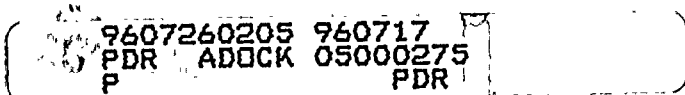
Submittal of Revised Technical Specification Pages for PG&E License
Amendment Request 96-01 -- Reconciliation With Received Amendment

References:

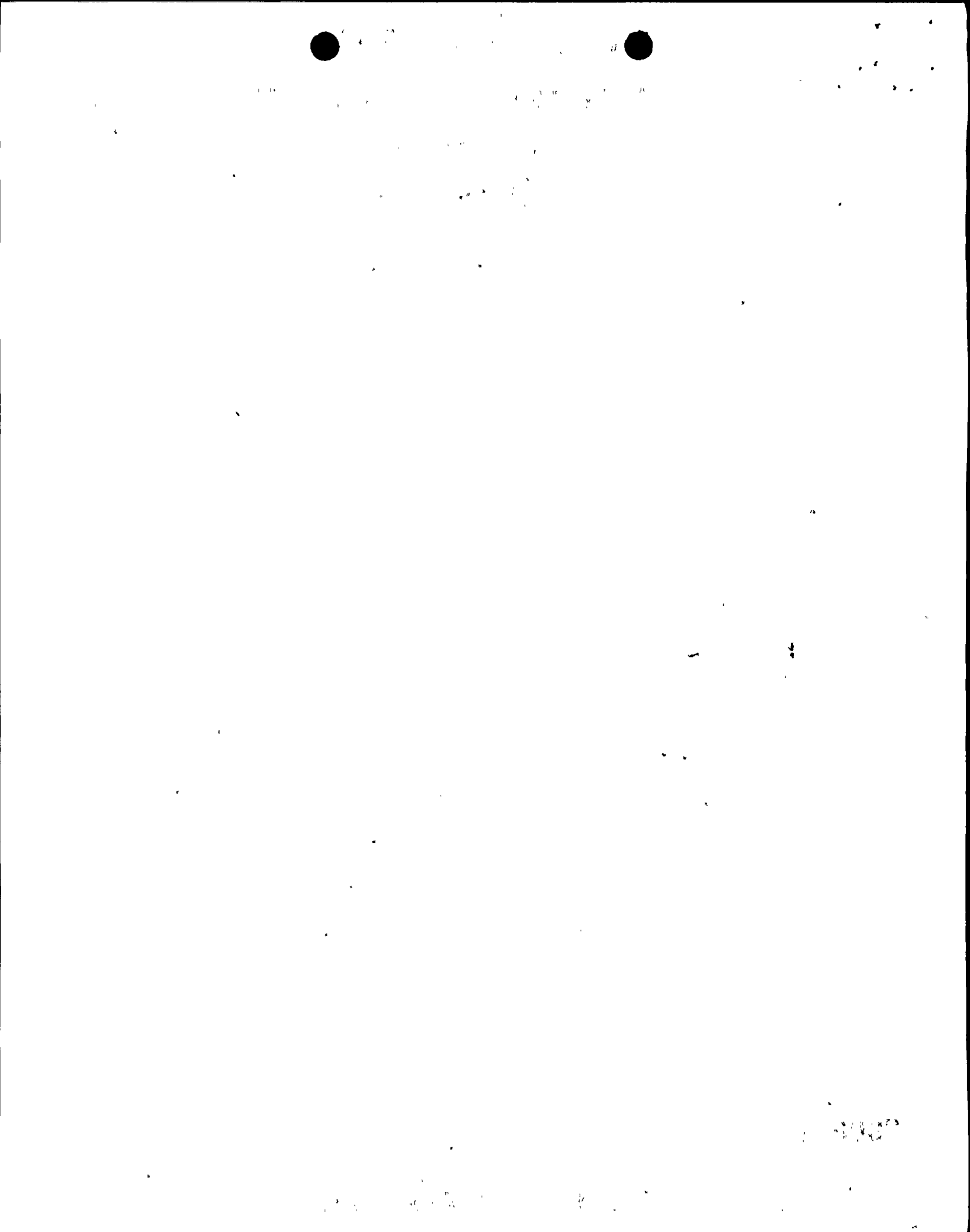
1. PG&E License Amendment Request 96-01, letter DCL-96-020, dated January 17, 1996, "Relocation of Selected Technical Specifications in Accordance with NRC Final Policy Statement and NUREG-1431, Rev. 1"
2. PG&E License Amendment Request 95-09, letter DCL-95-285, dated December 27, 1995, "Revision of Technical Specifications 3/4.6.1.1, 3/4.6.1.2, 3/4.6.1.3, 3/4.6.1.6, 3/4.6.3, Associated Bases, and Addition of 6.8.4j to Incorporate Requirements of the Containment Leakage Rate Testing Program -- Implementation of NRC Approved Regulation Revision, Option B, to 10 CFR 50, Appendix J"
3. NRC Letter dated March 1, 1996, "Issuance of Amendments for Diablo Canyon Nuclear Power Plant, Unit No. 1 (TAC No. M94379) and Unit No. 2 (TAC No. M94380)," License Amendments 110 (Unit 1)/109 (Unit 2)

Dear Commissioners and Staff:

Diablo Canyon Power Plant (DCPP) License Amendment Request (LAR) 96-01 (Ref. 1) proposed to relocate certain Technical Specifications (TS) consistent with the Commission's Final Policy Statement on TS Improvements, 10 CFR 50.36, and NUREG-1431, Rev 1. The proposed changes were based on TS pages that were current at the time of the submittal. One TS page has subsequently been amended.



Acc 1/1



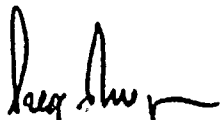
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Page 2

In Reference 2 PG&E submitted LAR 95-09, which requested changes to support implementation of performance based containment leakage rate testing as permitted by 10 CFR 50, Appendix J. By Reference 3 the NRC issued License Amendments 110 (Unit 1) and 109 (Unit 2) that implemented these changes. Included in these amendments was a change to a TS page that is also being proposed for revision by LAR 96-01.

This letter transmits a revised marked-up TS page in Enclosure 1 and a proposed new TS page in Enclosure 2 for the affected page of LAR 96-01. The enclosed page has been reconciled with DCP's current amended TS and contains no changes other than those requested in LAR 96-01. The page is a direct substitution for that originally submitted in LAR 96-01.

The revised TS page does not affect the conclusions of the safety evaluation or the no significant hazards consideration determination performed for LAR 96-01. The NRC priority for review and approval of this LAR should remain unchanged.

Sincerely,

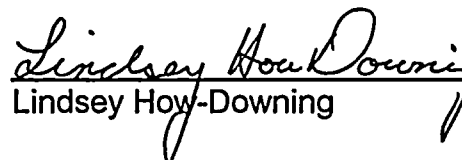


Gregory M. Rueger

Subscribed and sworn to before me
this 17th day of July 1996.



Attorneys for Pacific Gas and
Electric Company
Christopher J. Warner
Lindsey How-Downing



Lindsey How-Downing

cc: Edgar Bailey, DHS
Steven D. Bloom
L. J. Callan
Kenneth E. Perkins
Michael D. Tschiltz
Diablo Distribution

Enclosures



1952

1951

REVISED MARKED-UP TECHNICAL SPECIFICATIONS
FOR LAR 96-01 ATTACHMENT B

Instructions:

Remove Page from LAR

6-15c and accompanying insert A

Insert Page into LAR

Insert A (the insert will appear on
new page 6-15d when typed)



1954
1955
1956
1957

1958
1959
1960

Insert A
(new page 6-15d)

k. Explosive Gas and Storage Tank Radioactivity Monitoring Program

This program provides controls for potentially explosive gas mixtures contained in the Waste Gas Holdup System, the quantity of radioactivity contained in gas storage tanks, and the quantity of radioactivity contained in unprotected outdoor liquid storage tanks. The gaseous radioactivity quantities shall be determined following the methodology in Branch Technical Position (BTP) ETSB 11-5, "Postulated Radioactivity Release due to Waste Gas System Leak or Failure." The liquid radwaste quantities shall be determined in accordance with Standard Review Plan, Section 15.7.3, "Postulated Radioactive Release due to Tank Failures."

The program shall include:

- 1) The limits for the concentrations of hydrogen and oxygen in the Waste Gas Holdup System and a surveillance program to ensure the limits are maintained. Such limits shall be appropriate to the system's design criteria (i.e., whether or not the system is designed to withstand a hydrogen explosion);
- 2) A surveillance program to ensure that the quantity of radioactivity contained in each gas storage tank is less than the amount that would result in a whole body exposure of ≥ 0.5 rem to any individual in an UNRESTRICTED AREA, in the event of an uncontrolled release of the tanks' contents; and
- 3) A surveillance program to ensure that the quantity of radioactivity contained in all outdoor liquid radwaste tanks that are not surrounded by liners, dikes, or walls, capable of holding the tanks' contents and that do not have tank overflows and surrounding area drains connected to the Liquid Radwaste Treatment System, is less than the amount that would result in concentrations less than the limits of 10 CFR 20, Appendix B, Table II, Column 2, at the nearest potable water supply and the nearest surface water supply in an UNRESTRICTED AREA, in the event of an uncontrolled release of the tanks' contents.

The provisions of Specifications 4.0.2 and 4.0.3 are applicable to the Explosive Gas and Storage Tank Radioactivity Monitoring Program surveillance frequencies.



11-11-11
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