

Pacific Gas and Electric Company

77 Beale Street
San Francisco, CA 94106
415 973-4684
TAX 910-372-6587

James D. Shiffer
Senior Vice President and
General Manager
Nuclear Power Generator

June 28, 1990

PG&E Letter No. DCL-90-166



U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

Re: Docket No. 50-275, OL-DPR-80
Docket No. 50-323, OL-DPR-82
Diablo Canyon Units 1 and 2
Response to Generic Letter 90-04, Status of Implementation of
Generic Safety Issues (GSIs)

Gentlemen:

Generic Letter 90-04, "Request for Information on the Status of Licensee Implementation of Generic Safety Issues Resolved with Imposition of Requirements or Corrective Actions," dated April 25, 1990, requested that PG&E perform a review and report on the status of implementation of generic safety issues (GSIs). The enclosure provides the implementation status of each GSI for Diablo Canyon Units 1 and 2, references to documentation, and comments, as required, for clarification.

Kindly acknowledge receipt of this material on the enclosed copy of this letter and return it in the enclosed addressed envelope.

Sincerely,

A handwritten signature in cursive script, appearing to read 'J. D. Shiffer'. The signature is written in dark ink on a white background.

J. D. Shiffer

cc: A. P. Hodgdon
J. B. Martin
P. P. Narbut
S. A. Richards
H. Rood
CPUC
Diablo Distribution

Enclosure

3197S/0084K/DY/2232

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ENCLOSURE

STATUS OF IMPLEMENTATION OF GENERIC SAFETY ISSUES (GSIs)
FOR DIABLO CANYON UNITS 1 AND 2



FACILITY NAME: Diablo Canyon Power Plant Unit 1
 DOCKET NO.: 50-275, OL-DPR-80
 LICENSEE: Pacific Gas and Electric Company

STATUS OF LICENSEE IMPLEMENTATION OF GENERIC SAFETY ISSUES
RESOLVED WITH IMPOSITION OF REQUIREMENTS OR CORRECTIVE ACTIONS

<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
40 (B065)	Safety Concerns Associated With Pipe Breaks In The BWR Scram System	All BWRs	NA	
41 (B058)	BWR Scram Discharge Volume Systems	All BWRs	NA	
43 (B107)	Reliability Of Air Systems	All Plants	C	PG&E letters DCL-89-041, dated Feb. 21, 1989, and DCL-89-154, dated June 6, 1989, provided PG&E's response to Generic Letter 88-14. The letter provided PG&E's actions and schedule to implement the GL 88-14 recommendations. NRC letter dated June 12, 1989, provided acceptance of PG&E's actions regarding GL 88-14. GL 88-14 actions are completed, however, PG&E is continuing an effort to identify air system improvements for enhanced system performance. (Same status for Unit 2.)
51 (L913)	Improving the Reliability of Open-Cycle Service Water Systems	All Plants	I 12/91	PG&E letter DCL-90-027, dated Jan. 26, 1990, provided PG&E's plans and schedules regarding the establishment of programs to implement GL 89-13 recommendations. The commitments are being completed consistent with the schedule provided in DCL-90-027 with expected completion of the Unit 1-only actions during the 4th refueling outage of Unit 1 (May 1991) and expected completion of common Units 1 and 2 actions during the 4th refueling outage of Unit 2 (December 1991). (Same status for Unit 2.)
67.3.3 (A017)	Improved Accident Monitoring	All Plants	C	PG&E letters of Sept. 9, 1983, and DCL-84-298, dated Sept. 4, 1984 supplemented by letter, DCL-85-088, dated March 1, 1985 provided PG&E's response to Supplement 1 of NUREG-0737 summarizing compliance with Regulatory Guide 1.97, Rev. 3 for Unit 1. In letter DCL-86-086, dated March 27, 1986, PG&E provided information regarding the unavailability of acceptable radioactivity concentration monitors and boric acid charging flow meters. In PG&E letter DCL-90-037, dated Feb. 1, 1990, PG&E informed the NRC of installation of a qualified flowmeter in Unit 1. (Same status for Unit 2.)
75 (B076)	Item 1.1 - Post-Trip Review (Program Description and Procedure)	All Plants	NC	PG&E letters DCL-84-034, dated Feb. 1, 1984, and DCL-85-025, dated Jan. 24, 1985, provided PG&E's response to this item. NRC letter dated May 15, 1985, provided acceptance of PG&E's actions regarding this item. (Same status for Unit 2.)

* See Attachment for Coding Definition
 3197S/0084K-1



FACILITY NAME: Diablo Canyon Power Plant Unit 1
 DOCKET NO.: 50-275, OL-OPR-80
 LICENSEE: Pacific Gas and Electric Company

STATUS OF LICENSEE IMPLEMENTATION OF GENERIC SAFETY ISSUES

RESOLVED WITH IMPOSITION OF REQUIREMENTS OR CORRECTIVE ACTIONS

<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
75 (B085)	Item 1.2 - Post-Trip Review - Data and Information Capability	All Plants	NC	PG&E letter DCL-84-242, dated June 27, 1984, provided PG&E's response to this item. NRC letter dated June 25, 1984, provided acceptance of PG&E's actions regarding this item. (Same status for Unit 2.)
75 (B077)	Item 2.1 - Equipment Classification and Vendor Interface Data and Information Capability (Reactor Trip System Components)	All Plants	C	PG&E letters DCL-85-151, dated April 18, 1985, and DCL-87-057, dated Mar. 24, 1987, provided PG&E's response to this item. NRC letters dated Oct. 27, 1986 and July 24, 1987, provided acceptance of PG&E's actions regarding this item. (Same status for Unit 2.)
75 (B086)	Item 2.2.1 - Equipment Classification for Safety-Related Components	All Plants	C	PG&E letter DCL-87-156, dated June 30, 1987, provided PG&E's response to this item. NRC letter dated Oct. 30, 1987, provided acceptance of PG&E's actions regarding this item. (Same status for Unit 2.)
75 (L003)	Item 2.2.2 - Vendor Interface for Safety-Related Components	All Plants	E 9/90	PG&E letter DCL-87-156, dated June 30, 1987, provided PG&E's response to this item. Generic Letter 90-03 provided a relaxation of the NRC Staff's position on this item. PG&E's response is due to the NRC by Sept. 25, 1990. (Same status for Unit 2.)
75 (B078)	Items 3.1.1 & 3.1.2 - Post-Maintenance Testing (Reactor Trip System Components)	All Plants	NC	PG&E letters DCL-85-002, dated Jan. 2, 1985, and DCL-85-151, dated April 18, 1985, provided PG&E's response to these items. NRC letter dated April 10, 1986, provided acceptance of PG&E's actions regarding these items. (Same status for Unit 2.)
75 (B079)	Item 3.1.3 - Post-Maintenance Testing-Changes to Test Requirements (Reactor Trip System Components)	All Plants	NC	PG&E letters DCL-85-002, dated Jan. 2, 1985, and DCL-85-151, dated April 18, 1985, provided PG&E's response to this item. NRC letter dated June 26, 1986, provided acceptance of PG&E's actions regarding this item. (Same status for Unit 2.)
75 (B087)	Items 3.2.1 & 3.2.2 - Post-Maintenance Testing (All Other Safety-Related Components)	All Plants	C	PG&E letter DCL-86-074, dated March 20, 1986, provided PG&E's response to these items. NRC letter dated Oct. 21, 1986, provided acceptance of PG&E's actions regarding these items. (Same status for Unit 2.)
75 (B088)	Item 3.2.3 - Post-Maintenance Testing-Changes to Test Requirements (All Other Safety-Related Components)	All Plants	C	PG&E letter DCL-86-074, dated March 20, 1986, provided PG&E's response to this item. NRC letter dated June 26, 1986, provided acceptance of PG&E's actions regarding this item. (Same status for Unit 2.)

* See Attachment for Coding Definition
 3197S/0084K-2



FACILITY NAME: Diablo Canyon Power Plant Unit 1
 DOCKET NO.: 50-275, OL-DPR-80
 LICENSEE: Pacific Gas and Electric Company

STATUS OF LICENSEE IMPLEMENTATION OF GENERIC SAFETY ISSUES

RESOLVED WITH IMPOSITION OF REQUIREMENTS OR CORRECTIVE ACTIONS

<u>GSJ/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
75 (B080)	Item 4.1 - Reactor Trip System Reliability (Vendor-Related Modifications)	All Plants	NC	PG&E letter dated Nov. 7, 1983, provided PG&E's response to this item. NRC letter dated July 8, 1985, provided acceptance of PG&E's actions regarding this item. (Same status for Unit 2.)
75 (B081)	Items 4.2.1 & 4.2.2 - Reactor Trip System Reliability - Maintenance and Testing (Preventive Maintenance and Surveillance Program for Reactor Trip Breakers)	All PWRs	C	PG&E letters dated Nov. 7, 1983, DCL-85-070, dated Feb. 21, 1985, and DCL-85-151, dated April 18, 1985, provided PG&E's response to these items. DCL-88-132, dated May 17, 1988, provided additional information for Item 4.2.1. NRC letter dated June 24, 1985, provided acceptance of PG&E's actions regarding these items. (Same status for Unit 2.)
75 (B082)	Item 4.3 - Reactor Trip System Reliability - Design Modifications (Automatic Actuation of Shunt Trip Attachment for Westinghouse and B&W Plants)	All W and B&W Plants	C	PG&E letters dated Dec. 20, 1983, DCL-84-194, dated May 24, 1984, DCL-84-211, dated June 6, 1984, DCL-84-241, dated June 27, 1984, DCL-84-347, dated Nov. 7, 1984, DCL-86-336, dated Nov. 20, 1986, and DCL-88-298, dated Dec. 9, 1988, provided PG&E's response to this item. NRC letters dated May 16, 1984 and Jan. 6, 1989, provided acceptance of PG&E's actions regarding these items. (Same status for Unit 2.)
75 (B090)	Item 4.3 - Reactor Trip System Reliability - Tech Spec Changes (Automatic Actuation of Shunt Trip Attachment for Westinghouse and B&W Plants)	All W & B&W Plants	C	PG&E letters DCL-84-194, dated May 24, 1984, DCL-84-210, dated June 6, 1984, DCL-85-360, dated Dec. 6, 1985, DCL-88-307, dated Dec. 19, 1988, and DCL-89-074, dated Mar. 23, 1989, provided PG&E's response to this item. NRC letters dated May 16, 1984 and Jan. 6, 1989, provided acceptance of PG&E's actions regarding this item. (Same status for Unit 2.)
75 (B091)	Item 4.4 - Reactor Trip System Reliability (Improvements in Maintenance and Test Procedures for B&W Plants)	All B&W Plants	NA	
75 (B092)	Item 4.5.1 - Reactor Trip System Reliability-Diverse Trip Features (System Functional Testing)	All Plants	NC	PG&E letters DCL-84-241, dated June 27, 1984, DCL-84-347, dated Nov. 7, 1984, DCL-85-025, dated Jan. 24, 1985, and DCL-85-151, dated Apr. 18, 1985, provided PG&E's response to this item. NRC letter dated July 8, 1985, provided acceptance of PG&E's actions regarding this item. (Same status for Unit 2.)

* See Attachment for Coding Definition
 3197S/0084K-3



FACILITY NAME: Diablo Canyon Power Plant Unit 1
 DOCKET NO.: 50-275, OL-DPR-80
 LICENSEE: Pacific Gas and Electric Company

STATUS OF LICENSEE IMPLEMENTATION OF GENERIC SAFETY ISSUES
RESOLVED WITH IMPOSITION OF REQUIREMENTS OR CORRECTIVE ACTIONS

<u>GSJ/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
75 (B093)	Items 4.5.2 & 4.5.3 - Reactor Trip System Reliability - Test Alternatives and Intervals (System Functional Testing)	All Plants	NC	PG&E letters dated Nov. 7, 1983 and DCL-84-241, dated June 27, 1984, provided PG&E's response to Item 4.5.2. PG&E letter DCL-84-364, dated Nov. 30, 1984, provided PG&E's response to Item 4.5.3. NRC letter dated Mar. 18, 1987, provided acceptance of PG&E's actions regarding Item 4.5.2. NRC letter dated May 30, 1989, provided acceptance of PG&E's actions regarding Item 4.5.3. (Same status for Unit 2.)
86 (B084)	Long Range Plan for Dealing with Stress Corrosion Cracking in BWR Piping	All BWRs	NA	
93 (B098)	Steam Binding of Auxiliary Feedwater Pumps	All PWRs	C	PG&E letters DCL-86-047, dated Feb. 25, 1986 and DCL-88-135, dated May 23, 1988, responded to GL 85-01 and GL 88-03 respectively. NRC letter dated Feb. 23, 1989, indicated that PG&E's response to GL 88-03 was satisfactory and that procedure implementation had been verified by NRC Region V. (Same status for Unit 2.)
99 (L817)	RCS/RHR Suction Line Valve Interlock on PWRs	All PWRs	I 5/91	PG&E letter DCL 87-187, dated Aug. 4, 1987, submitted WCAP-11117 which provided justification for removal of the RHR System Autoclosure Interlock (ACI) Function. NRC letter dated February 17, 1988, provide the NRC Staff's safety evaluation and acceptance for removal of the ACI. Removal of the ACI and procedural enhancements have been completed for Unit 1. PG&E letter DCL-87-233, dated Sept. 18, 1987, responded to GL 87-12 regarding loss of RHR while the RCS is partially filled. PG&E letters DCL-89-005, dated Jan. 6, 1989, and DCL-89-030, dated Feb. 6, 1989, responded to GL 88-17 regarding loss of decay heat removal and expeditious actions and programmed enhancements. NRC letter dated April 26, 1989, provided acceptance of PG&E's expeditious actions and provided comments on PG&E's actions. Provision for Unit 1 RHR pump motor current trending is scheduled for installation during the fourth refueling outage (May 1991).

* See Attachment for Coding Definition
 3197S/0084K-4



FACILITY NAME: Diablo Canyon Power Plant Unit 1
 DOCKET NO.: 50-275, OL-DPR-80
 LICENSEE: Pacific Gas and Electric Company

STATUS OF LICENSEE IMPLEMENTATION OF GENERIC SAFETY ISSUES
RESOLVED WITH IMPOSITION OF REQUIREMENTS OR CORRECTIVE ACTIONS

<u>GSII(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
124	Auxiliary Feedwater System Reliability	ANO-1&2, Rancho Seco, Prairie Island 1&2, Crystal River-3, Ft. Calhoun	NA	
A-13 (B017)	Snubber Operability Assurance - Hydraulic Snubbers	All Plants	NC	The Units 1 and 2 Combined Technical Specifications, issued Aug. 26, 1985. Specification 3/4.7.7 provide operability and surveillance requirements for Diablo Canyon consistent with the Westinghouse Standard Technical Specifications. (Same status for Unit 2.)
A-13 (B022)	Snubber Operability Assurance - Mechanical Snubbers	All Plants	NC	The Units 1 and 2 Combined Technical Specifications, issued Aug. 26, 1985. Specification 3/4.7.7 provide operability and surveillance requirements for Diablo Canyon consistent with the Westinghouse Standard Technical Specifications. (Same status for Unit 2.)
A-16 (D012)	Steam Effects on BWR Core Spray Distribution	Oyster Creek & NMP-1	NA	
A-35 (B023)	Adequacy of Offsite Power Systems	All Plants	NC	PG&E letters dated Jan. 24, 1978, June 13, 1978, and Oct. 4, 1978, provided information regarding system protection for degraded voltage conditions. (Same status for Unit 2.)
B-10	Behavior of BWR Mark III Containments	All BWR Mark III Plants	NA*	
B-36	Develop Design, Testing and Maintenance Criteria for Atmosphere Cleanup System Air Filtration and Adsorption Units for Engineered Safety Features Systems and for Normal Ventilation Systems	All Plants with OL Applications After 4/1/80	NA	
B-63 (B045)	Isolation of Low Pressure Systems Connected to the Reactor Coolant System Pressure Boundary	All Plants	C	PG&E letters DCL-85-320, dated Oct. 11, 1985, DCL-86-033, dated Feb. 13, 1986, DCL-86-238, dated Aug. 12, 1986, and DCL-87-129, dated June 8, 1987, provided information regarding periodic testing of RCS pressure isolation valves in accordance with TS 4.4.6.2. NRC letter dated July 27, 1987, issued Amendment No. 16 for Unit 1 to include additional valves in TS Table 3.4-1. (Same status for Unit 2.)

* See Attachment for Coding Definition
 3197S/0084K-5



FACILITY NAME: Diablo Canyon Power Plant Unit 2
 DOCKET NO.: 50-323, OL-OPR-82
 LICENSEE: Pacific Gas and Electric Company

STATUS OF LICENSEE IMPLEMENTATION OF GENERIC SAFETY ISSUES
RESOLVED WITH IMPOSITION OF REQUIREMENTS OR CORRECTIVE ACTIONS

<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
40 (B065)	Safety Concerns Associated With Pipe Breaks In The BWR Scram System	All BWRs	NA	
41 (B058)	BWR Scram Discharge Volume Systems	All BWRs	NA	
43 (B107)	Reliability Of Air Systems	All Plants	C	PG&E letters DCL-89-041, dated Feb. 21, 1989, and DCL-89-154, dated June 6, 1989, provided PG&E's response to Generic Letter 88-14. The letter provided PG&E's actions and schedule to implement the GL 88-14 recommendations. NRC letter dated June 12, 1989, provided acceptance of PG&E's actions regarding GL 88-14. GL 88-14 actions are completed, however, PG&E is continuing an effort to identify air system improvements for enhanced system performance. (Same status for Unit 1.)
51 (L913)	Improving the Reliability of Open-Cycle Service Water Systems	All Plants	I 12/91	PG&E letter DCL-90-027, dated Jan. 26, 1990, provided PG&E's plans and schedules regarding the establishment of programs to implement GL 89-13 recommendations. The commitments are being completed consistent with the schedule provided in DCL-90-027 with expected completion of the Unit 2 and common Units 1 and 2 actions during the 4th refueling outage of Unit 2 (December 1991). (Same status for Unit 1.)
67.3.3 (A017)	Improved Accident Monitoring	All Plants	C	PG&E letters of Sept. 9, 1983, and DCL-84-298, dated Sept. 4, 1984 supplemented by letters DCL-85-024, dated Jan. 25, 1984, and DCL-85-243, dated July 18, 1985, provided PG&E's response to Supplement 1 of NUREG-0737 summarizing compliance with Regulatory Guide 1.97, Rev. 3 for Unit 2. In letter DCL-86-086, dated March 27, 1986, PG&E provided information regarding the unavailability of acceptable radioactivity concentration monitors and boric acid charging flow meters. In PG&E letter DCL-90-121, dated May 7, 1990, PG&E informed the NRC of installation of a qualified flowmeter in Unit 2. (Same status for Unit 1.)
75 (B076)	Item 1.1 - Post-Trip Review (Program Description and Procedure)	All Plants	NC	PG&E letters DCL-84-034, dated Feb. 1, 1984, and DCL-85-025, dated Jan. 24, 1985, provided PG&E's response to this item. NRC letter dated May 15, 1985, provided acceptance of PG&E's actions regarding this item. (Same status for Unit 1.)

* See Attachment for Coding Definition
 3197S/0084K-1



FACILITY NAME: Diablo Canyon Power Plant Unit 2
 DOCKET NO.: 50-323, OL-DPR-82
 LICENSEE: Pacific Gas and Electric Company

STATUS OF LICENSEE IMPLEMENTATION OF GENERIC SAFETY ISSUES
RESOLVED WITH IMPOSITION OF REQUIREMENTS OR CORRECTIVE ACTIONS

<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
75 (B085)	Item 1.2 - Post-Trip Review - Data and Information Capability	All Plants	NC	PG&E letter DCL-84-242, dated June 27, 1984, provided PG&E's response to this item. NRC letter dated June 25, 1984, provided acceptance of PG&E's actions regarding this item. (Same status for Unit 1.)
75 (B077)	Item 2.1 - Equipment Classification and Vendor Interface Data and Information Capability (Reactor Trip System Components)	All Plants	C	PG&E letters DCL-85-151, dated April 18, 1985, and DCL-87-057, dated Mar. 24, 1987, provided PG&E's response to this item. NRC letters dated Oct. 27, 1986 and July 24, 1987, provided acceptance of PG&E's actions regarding this item. (Same status for Unit 1.)
75 (B086)	Item 2.2.1 - Equipment Classification for Safety-Related Components	All Plants	C	PG&E letter DCL-87-156, dated June 30, 1987, provided PG&E's response to this item. NRC letter dated Oct. 30, 1987, provided acceptance of PG&E's actions regarding this item. (Same status for Unit 1.)
75 (L003)	Item 2.2.2 - Vendor Interface for Safety-Related Components	All Plants	E 9/90	PG&E letter DCL-87-156, dated June 30, 1987, provided PG&E's response to this item. Generic Letter 90-03 provided a relaxation of the NRC Staff's position on this item. PG&E's response is due to the NRC by Sept. 25, 1990. (Same status for Unit 1.)
75 (B078)	Items 3.1.1 & 3.1.2 - Post-Maintenance Testing (Reactor Trip System Components)	All Plants	NC	PG&E letter DCL-85-151, dated April 18, 1985, provided PG&E's response to these items. NRC letter dated April 10, 1986, provided acceptance of PG&E's actions regarding these items. (Same status for Unit 1.)
75 (B079)	Item 3.1.3 - Post-Maintenance Testing-Changes to Test Requirements (Reactor Trip System Components)	All Plants	NC	PG&E letter DCL-85-151, dated April 18, 1985, provided PG&E's response to this item. NRC letter dated March 16, 1987, provided acceptance of PG&E's actions regarding this item. (Same status for Unit 1.)
75 (B087)	Items 3.2.1 & 3.2.2 - Post-Maintenance Testing (All Other Safety-Related Components)	All Plants	C	PG&E letter DCL-86-227, dated Aug. 1, 1986, provided PG&E's response to this item. NRC letter dated Oct. 21, 1986, provided acceptance of PG&E's actions regarding this item. (Same status for Unit 1.)
75 (B088)	Item 3.2.3 - Post-Maintenance Testing-Changes to Test Requirements (All Other Safety-Related Components)	All Plants	C	PG&E letter DCL-86-227, dated Aug. 1, 1986, provided PG&E's response to this item. NRC letter dated March 16, 1987, provided acceptance of PG&E's actions regarding this item. (Same status for Unit 1.)

* See Attachment for Coding Definition
 3197S/0084K-2



FACILITY NAME: Diablo Canyon Power Plant Unit 2
 DOCKET NO.: 50-323, OL-DPR-82
 LICENSEE: Pacific Gas and Electric Company

STATUS OF LICENSEE IMPLEMENTATION OF GENERIC SAFETY ISSUES
RESOLVED WITH IMPOSITION OF REQUIREMENTS OR CORRECTIVE ACTIONS

<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
75 (B080)	Item 4.1 - Reactor Trip System Reliability (Vendor-Related Modifications)	All Plants	NC	PG&E letter dated Nov. 7, 1983, provided PG&E's response to this item. NRC letter dated July 8, 1985, provided acceptance of PG&E's actions regarding this item. (Same status for Unit 1.)
75 (B081)	Items 4.2.1 & 4.2.2 - Reactor Trip System Reliability - Maintenance and Testing (Preventive Maintenance and Surveillance Program for Reactor Trip Breakers)	All PWRs	C	PG&E letters dated Nov. 7, 1983, DCL-85-070, dated Feb. 21, 1985, and DCL-85-151, dated April 18, 1985, provided PG&E's response to these items. DCL-88-132, dated May 17, 1988, provided additional information for Item 4.2.1. NRC letter dated June 24, 1985, provided acceptance of PG&E's actions regarding these items. (Same status for Unit 1.)
75 (B082)	Item 4.3 - Reactor Trip System Reliability - Design Modifications (Automatic Actuation of Shunt Trip Attachment for Westinghouse and B&W Plants)	All W and B&W Plants	C	PG&E letters dated Dec. 20, 1983, DCL-84-194, dated May 24, 1984, DCL-84-211, dated June 6, 1984, DCL-84-241, dated June 27, 1984, DCL-84-347, dated Nov. 7, 1984, DCL-86-336, dated Nov. 20, 1986, and DCL-88-298, dated Dec. 9, 1988, provided PG&E's response to this item. NRC letters dated May 16, 1984 and Jan. 6, 1989, provided acceptance of PG&E's actions regarding these items. (Same status for Unit 1.)
75 (B090)	Item 4.3 - Reactor Trip System Reliability - Tech Spec Changes (Automatic Actuation of Shunt Trip Attachment for Westinghouse and B&W Plants)	All W & B&W Plants	C	PG&E letters DCL-84-194, dated May 24, 1984, DCL-85-360, dated Dec. 6, 1985, DCL-88-307, dated Dec. 19, 1988, and DCL-89-074, dated Mar. 23, 1989, provided PG&E's response to this item. NRC letters dated May 16, 1984 and Jan. 6, 1989, provided acceptance of PG&E's actions regarding this item. (Same status for Unit 1.)
75 (B091)	Item 4.4 - Reactor Trip System Reliability (Improvements in Maintenance and Test Procedures for B&W Plants)	All B&W Plants	NA	
75 (B092)	Item 4.5.1 - Reactor Trip System Reliability-Diverse Trip Features (System Functional Testing)	All Plants	NC	PG&E letters DCL-84-241, dated June 27, 1984, DCL-84-347, dated Nov. 7, 1984, DCL-85-025, dated Jan. 24, 1985, and DCL-85-151, dated Apr. 18, 1985, provided PG&E's response to this item. NRC letter dated July 8, 1985, provided acceptance of PG&E's actions regarding this item. (Same status for Unit 1.)

* See Attachment for Coding Definition
 3197S/0084K-3



FACILITY NAME: Diablo Canyon Power Plant Unit 2
 DOCKET NO.: 50-323, OL-DPR-82
 LICENSEE: Pacific Gas and Electric Company

STATUS OF LICENSEE IMPLEMENTATION OF GENERIC SAFETY ISSUES
RESOLVED WITH IMPOSITION OF REQUIREMENTS OR CORRECTIVE ACTIONS

<u>GSJ/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
75 (8093)	Items 4.5.2 & 4.5.3 - Reactor Trip System Reliability - Test Alternatives and Intervals (System Functional Testing)	All Plants	NC	PG&E letters dated Nov. 7, 1983 and DCL-84-241, dated June 27, 1984, provided PG&E's response to Item 4.5.2. PG&E letter DCL-84-364, dated Nov. 30, 1984, provided PG&E's response to Item 4.5.3. NRC letter dated Mar. 18, 1987, provided acceptance of PG&E's actions regarding Item 4.5.2. NRC letter dated May 30, 1989, provided acceptance of PG&E's actions regarding Item 4.5.3. (Same status for Unit 1.)
86 (8084)	Long Range Plan for Dealing with Stress Corrosion Cracking in BWR Piping	All BWRs	NA	
93 (8098)	Steam Binding of Auxiliary Feedwater Pumps	All PWRs	C	PG&E letters DCL-86-047, dated Feb. 25, 1986 and DCL-88-135, dated May 23, 1988, responded to GL 85-01 and GL 88-03 respectively. NRC letter dated Feb. 23, 1989, indicated that PG&E's response to GL 88-03 was satisfactory and that procedure implementation had been verified by NRC Region V. (Same status for Unit 1.)
99 (L817)	RCS/RHR Suction Line Valve Interlock on PWRs	All PWRs	C	PG&E letter DCL 87-187, dated Aug. 4, 1987, submitted WCAP-11117 which provided justification for removal of the RHR System Autoclosure Interlock (ACI) Function. NRC letter dated February 17, 1988, provide the NRC Staff's safety evaluation and acceptance for removal of the ACI. Removal of the ACI and procedural enhancements have been completed for Unit 2. PG&E letter DCL-87-233, dated Sept. 18, 1987, responded to GL 87-12 regarding loss of RHR while the RCS is partially filled. PG&E letters DCL-89-005, dated Jan. 6, 1989, and DCL-89-030, dated Feb. 6, 1989, responded to GL 88-17 regarding loss of decay heat removal and expeditious actions and programmed enhancements. NRC letter dated April 26, 1989, provided acceptance of PG&E's expeditious actions and provided comments on PG&E's actions.

* See Attachment for Coding Definition
 3197S/0084K-4



FACILITY NAME: Diablo Canyon Power Plant Unit 2
 DOCKET NO.: 50-323, OL-DPR-82
 LICENSEE: Pacific Gas and Electric Company

STATUS OF LICENSEE IMPLEMENTATION OF GENERIC SAFETY ISSUES
RESOLVED WITH IMPOSITION OF REQUIREMENTS OR CORRECTIVE ACTIONS

<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
124	Auxiliary Feedwater System Reliability	ANO-1&2, Rancho Seco, Prairie Island 1&2, Crystal River-3, Ft. Calhoun	NA	
A-13 (8017)	Snubber Operability Assurance - Hydraulic Snubbers	All Plants	NC	The Units 1 and 2 Combined Technical Specifications, issued Aug. 26, 1985. Specification 3/4.7.7 provide operability and surveillance requirements for Diablo Canyon consistent with the Westinghouse Standard Technical Specifications. (Same status for Unit 1.)
A-13 (8022)	Snubber Operability Assurance - Mechanical Snubbers	All Plants	NC	The Units 1 and 2 Combined Technical Specifications, issued Aug. 26, 1985. Specification 3/4.7.7 provide operability and surveillance requirements for Diablo Canyon consistent with the Westinghouse Standard Technical Specifications. (Same status for Unit 1.)
A-16 (0012)	Steam Effects on BWR Core Spray Distribution	Oyster Creek & NMP-1	NA	
A-35 (8023)	Adequacy of Offsite Power Systems	All Plants	NC	PG&E letters dated Jan. 24, 1978, June 13, 1978, and Oct. 4, 1978, provided information regarding system protection for degraded voltage conditions. (Same status for Unit 1.)
B-10	Behavior of BWR Mark III Containments	All BWR Mark III Plants	NA	
B-36	Develop Design, Testing and Maintenance Criteria for Atmosphere Cleanup System Air Filtration and Adsorption Units for Engineered Safety Features Systems and for Normal Ventilation Systems	All Plants with OL Applications After 4/1/80	NA	
B-63 (8045)	Isolation of Low Pressure Systems Connected to the Reactor Coolant System Pressure Boundary	All Plants	C	PG&E letters DCL-85-320, dated Oct. 11, 1985, DCL-86-033, dated Feb. 13, 1986, DCL-86-238, dated Aug. 12, 1986, and DCL-87-129, dated June 8, 1987, provided information regarding periodic testing of RCS pressure isolation valves in accordance with TS 4.4.6.2. NRC letter dated July 27, 1987, issued Amendment No. 15 for Unit 2 to include additional valves in TS Table 3.4-1. (Same status for Unit 1.)

* See Attachment for Coding Definition
 3197S/0084K-5



Attachment

1. NA - GSI is not applicable.
2. NC - GSI is applicable but no changes were necessary to implement the resolution, or the GSI implementation was completed prior to issuance of the operating license and no post-licensing changes were necessary.
3. C - GSI is applicable; submittal of information and/or changes were necessary and such submittals were made or changes are complete.
4. I - GSI is applicable and changes were necessary but such changes are not yet fully implemented.
5. E - Implementation guidance for a resolved GSI was issued recently and is still being evaluated.



ENCLOSURE 2

FACILITY NAME: Diablo Canyon Unit 1
 DOCKET NO.: 50-275
 LICENSEE: Pacific Gas and Electric Company

GSI Status Summary

<u>GSI/MPA No.</u>	<u>Title</u>	<u>Licensee Status Determination</u>	<u>Staff Status Determination</u>	<u>Remarks</u>
40 (B065)	Safety Concerns Associated With Pipe Breaks In The BWR Scram System	NA	NA	BWRs Only
41 (B058)	BWR Scram Discharge Volume Systems	NA	NA	BWRs Only
43 (B107)	Reliability Of Air Systems	C (06/90)	C (06/90)	Modifications to meet GL 88-14 are complete.
51 (L913)	Improving the Reliability of Open-Cycle Service Water Systems	I (12/91)	I (12/91)	Unit 1 modifications to be completed 05/91. Unit 2 and common area modifications to be completed 12/91.
67.3.3 (A017)	Improved Accident Monitoring	C (02/90)	I	Qualified flowmeter installed. Staff position on this issue is being re-evaluated.
75 (B076)	Item 1.1 - Post-Trip Review (Program Description and Procedure)	NC	NC	No hardware changes required.



<u>GSI/MPA No.</u>	<u>Title</u>	<u>Licensee Status Determination</u>	<u>Staff Status Determination</u>	<u>Remarks</u>
75 (B085)	Item 1.2 - Post-Trip Review - Data and Information Capability	NC	NC	
75 (B077)	Item 2.1 - Equipment Classification and Vendor Interface (Reactor Trip System Components)	C (03/87)	C (03/87)	
75 (B086)	Item 2.2.1 - Equipment Classification for Safety-Related Components	C (06/87)	C (06/87)	
75 (L003)	Item 2.2.2 - Vendor Interface for Safety-Related Components	E (09/90)	E (09/90)	PG&E response to GL 90-03 is scheduled to be submitted on September 25, 1990.
75 (B078)	Items 3.1.1 & 3.1.2 - Post - Maintenance Testing (Reactor Trip System Components)	NC	NC	



<u>GSI/MPA No.</u>	<u>Title</u>	<u>Licensee Status Determination</u>	<u>Staff Status Determination</u>	<u>Remarks</u>
75 (B079)	Item 3.1.3 - Post-Maintenance Testing - Changes to Test Requirements (Reactor Trip System Components)	NC	NC	
75 (B087)	Items 3.2.1 & 3.2.2 - Post-Maintenance Testing (All Other Safety-Related Components)	C (03/86)	C (03/86)	
75 (B088)	Item 3.2.3 - Post-Maintenance Testing-Changes to Test Requirements (All Other Safety-Related Components)	C (03/86)	C (03/86)	
75 (B080)	Item 4.1 - Reactor Trip System Reliability (Vendor-Related Modifications)	NC	NC	



<u>GSI/MPA No.</u>	<u>Title</u>	<u>Licensee Status Determination</u>	<u>Staff Status Determination</u>	<u>Remarks</u>
75 (B081)	Items 4.2.1 & 4.2.2 - Reactor Trip System Reliability - Maintenance and Testing (Preventative Maintenance and Surveillance Program for Reactor Trip Breakers)	C (05/88)	C (05/88)	
75 (B082)	Item 4.3 - Reactor Trip System Reliability - Design Modifications (Automatic Actuation of Shunt Trip Attachment for Westinghouse and B&W Plants)	C (12/88)	C (12/88)	
75 (B090)	Item 4.3 - Reactor Trip System Reliability - Tech Spec Changes (Automatic Actuation of Shunt Trip Attachment For Westinghouse and B&W Plants)	C (03/89)	C (03/89)	



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<u>GSI/MPA No.</u>	<u>Title</u>	<u>Licensee Status Determination</u>	<u>Staff Status Determination</u>	<u>Remarks</u>
75 (B091)	Item 4.4 - Reactor Trip System Reliability (Improvements in Maintenance and Test Procedures for B&W Plants)	NA	NA	B&W plants only.
75 (B092)	Item 4.5.1 - Reactor Trip System Reliability-Diverse Trip Features (System Functional Testing)	NC	NC	
75 (B093)	Items 4.5.2 & 4.5.3 - Reactor Trip System Reliability - Test Alternatives and Intervals (System Functional Testing)	NC	NC	
86 (B084)	Long Range Plan for Dealing with Stress Corrosion Cracking in BWR Piping	NA	NA	BWRs only.



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<u>GSI/MPA No.</u>	<u>Title</u>	<u>Licensee Status Determination</u>	<u>Staff Status Determination</u>	<u>Remarks</u>
93 (B098)	Steam Binding of Auxiliary Feedwater Pumps	C (05/88)	C (05/88)	
99 (L817)	RCS/RHR Suction Line Valve Interlock on PWRs	I (05/91)	I (05/91)	Provisions for RHR pump motor current trending will be installed 05/91.
124	Auxiliary Feedwater System Reliability	NA	NA	Not applicable to plants with three AFW pumps.
A-13 (B017)	Snubber Operability Assurance - Hydraulic Snubbers	NC	NC	Covered in combined tech specs issued with Unit 2 full power operating license (08/26/85).
A-13 (B022)	Snubber Operability Assurance - Mechanical Snubbers	NC	NC	Covered in combined tech specs issued with Unit 2 full power operating license (08/26/85).
A-16 (D012)	Steam Effects on BWR Core Spray Distribution	NA	NA	BWRs only.
A-35 (B023)	Adequacy of Offsite Power Systems	NC	NC	

<u>GSI/MPA No.</u>	<u>Title</u>	<u>Licensee Status Determination</u>	<u>Staff Status Determination</u>	<u>Remarks</u>
B-10	Behavior of BWR Mark III Containments	NA	NA	BWR Mark III plants only.
B-36	Develop Design, Testing and Maintenance Criteria for Atmosphere Cleanup System Air Filtration and Adsorption Units for Engineered Safety Features Systems and for Normal Ventilation Systems	NA	NA	Not applicable to plants with OL application tendered after 04/01/80. Diablo Canyon OL application tendered in 1973.
B-63 (B045)	Isolation of Low Pressure Systems Connected to the Reactor Coolant System Pressure Boundary	C (07/87)	C (07/87)	

