

CENSEE EVENT REPORT (LER)

FACILITY NAME (1): **DIABLO CANYON UNIT 1** DOCKET NUMBER (2): **051010121715** PAGE (3) **1** OF **011**

TITLE (4): **POTENTIALLY NONCONSERVATIVE ASSUMPTIONS USED IN FSAR LOSS OF LOAD/TURBINE TRIP ANALYSIS**

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	MONTH	DAY	YEAR	MONTH	DAY	YEAR	FACILITY NAME(S)	DOCKET NUMBER(S)
07	17	87	87	01	10	01	08	21	87	Diablo Canyon Unit 2	051010131213
											0510101111

OPERATING MODE (9): **1** THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § 111

POWER LEVEL (10): **1010**

10 CFR _____

OTHER (Specify in Abstract below and in Test, NRC Form 305A) **VOLUNTARY**

LICENSEE CONTACT FOR THIS LER (12):

STEPHEN D. WILSON, REGULATORY COMPLIANCE ENGINEER

TELEPHONE NUMBER: **8105 51951-171311**

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14): YES (15) OR UNKNOWN EXPECTED SUBMISSION DATE: NO

EXPECTED SUBMISSION DATE (16):

ABSTRACT (18):

This report is being voluntary submitted for information purposes only as described in Item 19 of Supplement Number 1 to NUREG-1022.

On July 17, 1987, PGandE received Westinghouse Electric Corporation letter PGE-87-092, dated July 9, 1987, regarding a generic concern involving potentially nonconservative assumptions used in the FSAR analysis for the loss of external electrical load and/or turbine trip event.

The Westinghouse scenario postulates that a reactor trip does not occur in the first 30 seconds following the turbine trip (the period during which the generator is motored). A complete loss of forced reactor coolant flow could be assumed to occur 30 seconds into the transient due to a postulated single failure in the fast bus transfer to offsite power. The postulated loss of flow could occur at essentially full thermal power with the reactor at off-normal conditions (i.e., vessel inlet temperature approximately 15°F above nominal T-inlet). This leads to the possibility of exceeding the departure from nucleate boiling ratio (DNBR) design limit.

PGandE has evaluated the scenario postulated by Westinghouse and has determined that it does not present a significant safety hazard. This determination is based upon the low probability of the event occurring, the high reliability of the reactor trip on turbine trip function, and the likelihood that this event is bounded by the complete loss of flow FSAR analysis. PGandE will monitor the Westinghouse Owners Group (WOG) effort for a long-term resolution of this generic concern.

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PACIFIC GAS AND ELECTRIC COMPANY

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JAMES D. SHIFFER
VICE PRESIDENT
NUCLEAR POWER GENERATION

August 21, 1987

PGandE Letter No: DCL-87-208

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

Re: Docket No. 50-275, OL-DPR-80
Docket No. 50-323, OL-DPR-82
Diablo Canyon Units 1 and 2
Licensee Event Report 1-87-010-00 (Voluntary)
Potentially Nonconservative Assumptions Used in FSAR Loss of
Load/Turbine Trip Analysis

Gentlemen:

PGandE is submitting the enclosed voluntary Licensee Event Report regarding potentially nonconservative assumptions used in the FSAR analysis for the loss of load/turbine trip event. This report is being submitted for information purposes only as described in Item 19 of Supplement Number 1 to NUREG-1022.

Kindly acknowledge receipt of this material on the enclosed copy of this letter and return it in the enclosed addressed envelope.

Sincerely,
J. D. Shiffer
J. D. Shiffer

Enclosure

cc: L. J. Chandler
J. B. Martin
M. H. Mendonca
P. P. Narbut
B. Norton
CPUC
Diablo Distribution

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