

UNITED STATES NUCLEAR REGULATORY COMMISSION ADVISORY COMMITTEE ON REACTOR SAFEGUARDS WASHINGTON, D. C. 26555

October 13, 1982

Mr. William J. Dircks
Executive Director for Operations
U. S. Nuclear Regulatory Commission
Washington, DC 20555

Dear Mr. Dircks:

SUBJECT:

ACRS COMMENTS ON NRC PROGRAM TO ADDRESS CONCERNS WITH THERMAL HYDRAULIC BEHAVIOR OF BASCOCK AND WILCOX PLANTS DURING TRANSIENTS AND ACCIDENTS -

During its 270th Meeting, October 7-8, 1982, the Advisory Committee on Reactor Safeguards met with the NRC Staff and representatives of the Babcock and Wilcox (B&W) Owners Group to discuss NRC Staff concerns regarding the dynamic thermal hydraulic behavior of B&W plants during transients and accidents, particularly small break loss of coolant accidents.

For some time, the NRC Staff has identified a need for experimental data for investigation of specific plant phenomena and for assessment of analytical calculations of B&W plant response to transients and accidents. Recently, a Test Advisory Group composed of NRC Staff members and representatives of the B&W Owners Group was formed to evaluate alternatives available for obtaining the desired test data. The Owners Group has proposed use of two industry test facilities (GERDA and SRI-II) in response to the NRC Staff's concerns.

While we support the cooperative effort between NRC and the Owners Group, it appears that the GERDA and SRI-II facilities as now proposed will be inadequate to satisfactorily address the NRC Staff concerns in this matter. Although the data obtained from these facilities may be useful, we believe that a more adequate facility, similar to the proposed Semiscale MOD-5 configuration, is necessary to address the major operational questions of concern. We also wish to emphasize that the timely acquisition of such data and associated analyses are required in order that NRR can make use of S&W plant accident analyses confidently.

Sincerely,

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